

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Docket No. 50-282  
50-306

REQUEST FOR AMENDMENT TO  
OPERATING LICENSE NOS. DPR-42 & DPR-60

(License Amendment Request Dated April 5, 1985)

Northern States Power Company, a Minnesota corporation, requests authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

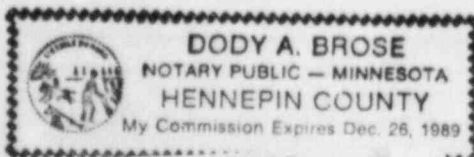
This letter contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By David Musolf  
David Musolf  
Manager - Nuclear Support Services

On this 5th day of April, 1985 before me a notary public in and for said County, personally appeared David Musolf, Manager - Nuclear Support Services, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.

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Exhibit A

Prairie Island Nuclear Generating Plant

License Amendment Request Dated April 5, 1985

Evaluation of Proposed Changes to the  
Technical Specifications Appendix A of  
Operating License DPR-42 and DPR-60

Pursuant to 10 CFR Part 50, Sections 50.59 and 50.90 the holders of Operating Licenses DPR-42 and DPR-60 hereby propose the following changes to Appendix A, Technical Specifications:

Specification: Shunt Trip - TS 3.5/4.1

Proposed Change

Technical Specification Sections 3.5 and 4.1 are being modified to incorporate operability and testing requirements associated with the shunt trip attachment modification required by Generic Letter 83-28. The specific changes are provided in Exhibit B.

Reason For Change

In response to Generic Letter 83-28, "Required Actions Based on Generic Implications of Salem ATWS Events", modifications are being made at Prairie Island to provide automatic reactor trip system actuation of the reactor trip breaker shunt trip attachments. The NRC Generic Safety Evaluation Report for the Westinghouse Shunt Trip Modification, dated August 10, 1983, requested Technical Specification changes that provide for periodic testing of the shunt trip attachments and manual reactor trip circuits. The proposed technical specification changes are being submitted in response to that request.

Significant Hazards Evaluation

Preliminary technical specifications, incorporating the shunt trip and manual reactor trip requirements requested by the August 10, 1983 Generic SER, were provided to the Commission in our December 10, 1984 letter. Those proposed technical specifications, with some additions and minor wording changes, were found to be acceptable by the Commission in the Reactor Trip Breaker Automatic Shunt Trip Safety Evaluation Report dated February 21, 1985.

Because the Commission has already found the proposed technical specifications to be acceptable, and because the proposed changes constitute additional limitations, restrictions, or controls not presently included in the technical specifications, operation of the Prairie Island Nuclear Generating Plant in accordance with these proposed changes will not:

- (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) involve a significant reduction in a margin of safety.

#### Schedule for Implementation

The shunt trip modification is complete in Unit 1. Unit 2 will be modified during the next refueling outage. We request these Technical specification changes be issued with a delayed implementation of prior to Unit 2 Cycle 10 operation.