NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Callaway Plant Unit 1	0 5 0 0 0 4 8 3	1 OF 0 2

TITLE (4)
Inadvertent Engineered Safety Features Actuation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 0	3 0	8 4	8 4	0 5 8	0 0	1 1	2 9	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 3 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Charles D. Naslund - Superintendent, I&C	AREA CODE 3 1 1 4 6 7 6 1 - 8 5 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/30/84 at 1152 CST, with plant in Mode 1 at 30% power, Feedwater Isolation (FWIS), Auxiliary Feedwater Actuation (AFAS) and Steam Generator Blowdown Isolation (SGBIS) signals were initiated by a high level on steam generator (S/G) 'B.' The high level resulted from a transient initiated when the main turbine tripped on a spurious high vibration signal at 1150. Following the turbine trip, the steam dumps to the condenser failed to operate properly in the Tave mode, causing the S/G atmospheric reliefs to open. The oscillations between the atmospherics and the condenser dumps caused S/G level oscillations which reached the high level setpoint on S/G 'B.'

The steam dumps were switched to the steam pressure mode, allowing the atmospherics to close. Reactor power was reduced and stabilized at 3% (Mode 2). the FWIS was reset at 1200, the AFAS reset at 1220 and the SGBIS reset at 1245.

This event posed no threat to the public health or safety.

IE22
1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Callaway Plant Unit 1	0 5 0 0 0 4 8 3	8 4	— 0 5 8	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 10/30/84 at 1150 CST, with plant in Mode 1 at 30% Reactor power and main generator output at 250 MW, the main turbine tripped on a spurious high vibration signal. Following the turbine trip, the steam dumps to the condenser failed to operate properly in the Tave (automatic) mode, causing the steam generator (S/G) atmospheric reliefs to open. The oscillations between the atmospherics and the condenser dumps caused S/G level oscillations which reached the high level setpoint on S/G 'B' at 1152, causing Feedwater Isolation (FWIS), Auxiliary Feedwater Actuation (AFAS) and S/G Blowdown Isolation (SGBIS).

Operators placed the steam dumps in the steam pressure mode and opened the dumps, allowing the atmospherics to close. Reactor power was reduced and stabilized at 3% (Mode 2). The FWIS was reset at 1200, the AFAS reset at 1220 and the SGBIS reset at 1245.

Investigation of the vibration on the main turbine revealed nothing abnormal and was therefore determined to be a spurious trip. The operation of the steam dumps was investigated and the problem was determined to be improper wiring termination from the turbine impulse chamber which disabled steam dump permissive C-7 and prevented the automatic operation of the dumps when the turbine tripped. This problem had not been identified by previous testing.

The steam dump wiring termination problem was corrected and the dumps verified to be operating properly.

This event posed no threat to the public health or safety.

Previous occurrences: none

UNION ELECTRIC COMPANY
CALLAWAY PLANT

MAILING ADDRESS:
P.O. BOX 620
FULTON, MO. 65251

November 29, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-987

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-058-00
INADVERTENT ENGINEERED SAFETY FEATURES ACTUATION

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning inadvertent Engineered Safety Features actuation.

Andrew P. Neuhalfen

for S. E. Miltenberger
Manager, Callaway Plant

CDN/WRR/DRM/drs
Enclosure

cc: Distribution attached

IE22
1/1

cc distribution for ULNRC-987

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N. Date