

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
LaSalle County Station Unit 1	0 5 0 0 0 3 1 7 1 3	1 OF 0 3

MECHANICAL SNUBBER FOUND LOCKED DURING ROUTINE SURVEILLANCE

EVENT DATE (6)			LEA NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)				
10	10	84	84	063	00	10	31	84	NA					05000				
														05000				

OPERATING MODE (S)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 16 CFR § (Check one or more of the following) (11)									
4		25.402(a)		25.405(a)		25.734(a)(2)(iv)		73.71(b)			
POWER LEVEL (W)		25.405(a)(1)(i)		25.734(a)(1)		25.734(a)(2)(v)		73.71(c)			
0 0 0		25.405(a)(1)(ii)		25.734(a)(2)		25.734(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 395A)			
		25.405(a)(1)(iii)		25.734(a)(2)(i)		25.734(a)(2)(vii)(A)					
		25.405(a)(1)(iv)	X	25.734(a)(2)(ii)		25.734(a)(2)(vii)(B)					
		25.405(a)(1)(v)		25.734(a)(2)(iii)		25.734(a)(2)(viii)					

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME		AREA CODE	
Daniel R. Szumski, extension 447		8115	3571-167611

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS	
X	A D	S N B	P O 2 9	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)					
<input checked="" type="checkbox"/>	NO				

ABSTRACT (Limit is 1400 words, i.e., approximately fifteen single-space typewritten lines) (18)

A visual inspection of snubbers normally inaccessible during power operations revealed a damaged unit on the "A" reactor recirculation pump seal purge line. The snubber was unpinned to be hand stroked to determine operability, but it was found to be locked up and rigid. Damage to the snubber indicated that the unit had been stepped on. There had been very heavy work activity around this snubber prior to inspection associated with the replacement of the recirculation pump seal. The snubber passed a visual inspection in February, 1984. The additional stresses on the purge line under thermal and seismic loading conditions had the potential to compromise the physical integrity of the piping. The loss of the seal purge fluid would cause elevated seal temperatures, and eventual degradation of the pump seals.

The failed snubber was replaced, and the pipe analyzed for the thermal cycles seen since February, 1984. The piping was not determined to be degraded due to the snubber failure. A letter from the Station Superintendent to all Department Heads was written regarding proper work practices on and around snubbers to prevent recurrence of this type of failure.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 306A's) (17)

I. EVENT DESCRIPTION

A visual exam of the safety related snubbers which are inaccessible during power operations was performed during a unit maintenance outage and this exam revealed a damaged PSA 1/4 on the "A" reactor recirculation (AD) pump seal purge line. The unit was determined to be in a failed state after an attempt to unpin and stroke the snubber over its travel range showed it to be locked in a rigid condition.

II. CAUSE

The orientation of the snubber with respect to the clamp for attaching it to the system piping indicated that the snubber had been stepped on or used as a hand hold. Supporting this conclusion were: 1) The fact that the snubber passed a similar visual examination in February, 1984. 2) The unit had operated at high power levels all summer until the scheduled outage in September, 1984, and access to the snubber was thereby limited. 3) Immediately prior to the visual examination of the snubber, there was a heavy period of maintenance activity immediately around this snubber associated with the replacement of the recirculation pump seal.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

Having a snubber on the recirculation pump seal purge line act as a rigid restraint could cause a possible overstress situation to develop under expected seismic or thermal load conditions, which could lead to the loss of physical integrity of the piping. Loss of seal purge fluid during full power operations would lead to elevated pump seal temperatures in the short term, and eventual degradation of the seals in the long term. This line, however, is not part of the reactor coolant pressure boundary.

IV. CORRECTIVE ACTIONS

The piping the snubber was attached to was visually examined for loss of physical integrity, as was the auxiliary steel associated with the support, and no indications of damage were noted. A new PSA 1/4 type snubber was placed into service for the damaged unit on the seal purge line under Station Work Request L42276 on 10/10/84. An engineering evaluation was performed on the piping by Sargent & Lundy Engineers. The evaluation looked at the effect the rigid support had on the seal purge line during the thermal cycles the line may have seen between February, 1984, and unit shutdown in September, 1984. This was done to confirm that no overstress of the piping would have occurred should the snubber have failed immediately following its inspection in February. The analysis confirmed that the line was not damaged or degraded due to the additional stresses induced by the locked up snubber. The replacement of the failed snubber and the analysis of the piping met the requirements specified in Technical Specification Section 4.7.9.c for corrective action following a snubber failure during a visual surveillance. In addition, a letter was generated from the Station Superintendent to all station management personnel giving guidance on how to work on and around snubbers and requesting adherence to the guidelines presented to prevent similar failures in the future.

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V. PREVIOUS OCCURRENCES

LER-82-005/03L-0 (Inboard Main Steam Line Drain Snubber)
LER-82-036/03L-0 (Inboard Main Steam Line Drain Snubber)
LER-83-009/03L-0 (Outboard Main Steam Line Drain Snubber)

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Daniel R. Szumski, 815/357-6761, extension 447.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

October 31, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-063-00, Docket #050-373 is being submitted to your office in accordance with 10CFR 50.73.

G. J. Diederich 11/6/84
G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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