

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

February 15, 1996
LIC-96-0016

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

Reference: Docket No. 50-285

SUBJECT: January 1996 Monthly Operating Report (MOR)

Enclosed please find the January 1996 MOR for Fort Calhoun Station (FCS)
Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,



T. L. Patterson
Division Manager
Nuclear Operations

TLP/d11

Enclosures

c: Winston & Strawn
L. J. Callan, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. J. Simon, Westinghouse
INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

JANUARY 1996
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of January 1996, the Fort Calhoun Station (FCS) operated at a nominal 100% power. Normal plant maintenance, surveillance, equipment rotation activities and scheduled on-line modifications were performed during the month. Monitoring of a minor Control Element Drive Mechanism (CEDM) mechanical seal leak continued.

On January 19th, an ice jam formed upstream of Sioux City, Iowa, causing Missouri River levels to lower approximately five feet over a 24-hour period. Preparations were made to respond to potential icing or low river level conditions. On January 19th, the low river level Abnormal Operating Procedure (AOP-01) was implemented when it was projected that the river level would drop below 983 feet above sea level. As required by the procedure, the U. S. Army Corps of Engineers was requested to increase releases from Gavins Point Dam on January 20th. The Corps could not comply because flooding would occur upstream of the ice jam. Pursuant to 10CFR50.72(b)(2)(vi), a four-hour non-emergency notification was subsequently made to the NRC to report the off-site notification made to the U.S. Army Corps of Engineers. The FCS Technical Specifications require the reactor to be placed in cold shutdown if the river level is less than 976 feet, 9 inches. The lowest actual river level observed at FCS during this event was 982 feet, 6 inches with the level returning to normal after two days.

On January 24th, approximately 50 gallons of hydrazine leaked from a hydrazine storage container into the Turbine Building. Hazardous Materials personnel responded to isolate and clean up the spill. The spill exceeded the State limit, therefore, a notification was made to the State of Nebraska. A four-hour non-emergency report was made to the NRC pursuant to 10CFR50.72(b)(2)(vi).

Several of the incore nuclear detectors have recently failed rendering 2 of the 28 strings of detectors inoperable. These failures occurred in new detectors that were installed during the 1995 refueling outage. These failures are under investigation with assistance from ABB/CE and the detector vendor.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of January, no power operated relief valves (PORV) or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

Although above normal, the January Reactor Coolant System (RCS) leak rate was steady at approximately 0.30 gpm throughout the month. This leak rate has remained relatively steady following the reactor trip on August 26, 1995. Several increases in the leak rate were related to charging pump packing leaks during the month. Two charging pumps were repacked during January 1996 which returned the leak rate to a nominal 0.25 - 0.30 gpm.

The major contributor of the RCS leakage is "Known" leakage which is being collected in both the Reactor Coolant Drain Tank (RCDT) and the Pressurizer Quench Tank. The primary leakage source for "Known" leakage has been attributed to CEDM #15. The "Known" leak rate has decreased over the last several months from 0.28 gpm to approximately 0.15 gpm. In addition to CEDM #15, several of the reactor head vent system isolation valves are also leaking slightly with leakage being collected both in the Containment Sump and the Pressurizer Quench Tank. Leakage to the Containment Sump is considered "Unknown" leakage.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
None	

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF JANUARY 1996

- Due to the possibility of the piston and actuator shaft being mis-aligned, the Bettis Actuators on certain inlet/outlet isolation valves on the Component Cooling Water (CCW) Heat Exchangers were inspected and rebuilt as necessary to ensure proper alignment.
- The tubes on CCW Heat Exchanger AC-1C were cleaned and the associated Raw Water Inlet Valve HCV-2882A was rebuilt.
- Charging Pumps CH-1B and CH-1C were repacked due to excessive RCS leakage.
- Raw Water Pump AC-10B was replaced with a rebuilt pump assembly.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE FEBRUARY 05, 1996
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION

2. Reporting Period: JANUARY 1996

NOTES

3. Licensed Thermal Power (MWt): 1500

4. Nameplate Rating (Gross MWe): 502

5. Design Elec. Rating (Net MWe): 478

6. Max. Dep. Capacity (Gross MWe): 502

7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	744.0	195938.0
12. Number of Hours Reactor was Critical	744.0	744.0	154452.0
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	744.0	152724.5
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113114.0	1113114.0	203799422.3
17. Gross Elec. Energy Generated (MWH)..	379148.0	379148.0	67312873.2
18. Net Elec. Energy Generated (MWH)....	362257.6	362257.6	64219626.4
19. Unit Service Factor.....	100.0	100.0	77.9
20. Unit Availability Factor.....	100.0	100.0	77.9
21. Unit Capacity Factor (using MDC Net)	101.9	101.9	70.8
22. Unit Capacity Factor (using DER Net)	101.9	101.9	69.2
23. Unit Forced Outage Rate.....	.0	.0	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
A MAINTENANCE OUTAGE IS SCHEDULED TO OCCUR FROM MARCH 16-23, 1996 TO RE-PAIR/REPLACE DEGRADING CEDM MECHANICAL SEALS.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	FEBRUARY 05, 1996
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH JANUARY 1996

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	487
2	487
3	487
4	487
5	486
6	486
7	486
8	487
9	487
10	487
11	487
12	487
13	487
14	487
15	487
16	487

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

17	487
18	487
19	487
20	487
21	487
22	487
23	487
24	487
25	487
26	487
27	487
28	487
29	486
30	486
31	486

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE February 5, 1996
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH January 1996

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
NONE									

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun Station - Unit No. 1

Report for the month ending January 31, 1996

1. Scheduled date for next refueling shutdown. September 21, 1996
2. Scheduled date for restart following refueling. November 2, 1996
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be? Enrichment limit of spent fuel racks is to be increased to at least 4.5 w/o from 4.2 w/o. This is necessary based upon the preliminary Cycle 17 core pattern development.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. N/A
 - c. If no such review has taken place, when is it scheduled? N/A
4. Scheduled date(s) for submitting proposed licensing action and support information. Spent fuel rack enrichment limit change was submitted February 1, 1996
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. N/A
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 618 Assemblies
 - c) spent fuel pool storage capacity 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 2007 Outage

Prepared by Jan Bostelman

Date 2/5/96