

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) PALISADES NUCLEAR PLANT										DOCKET NUMBER (2) 0 5 0 0 0 2 5 5 1 OF 0 1										PAGE (3) 1																			
TITLE (4) Low Pressure Safety Injection Control Valve (CV-3006) Not Fully Open																																							
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																					
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)						
																											NA						0 5 0 0 0						
0 9 0			3 8 4			8 4			0 1 7			0 1 1			1 1 3			0 8 4			NA						0 5 0 0 0												
OPERATING MODE (9) N						THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																	
POWER LEVEL (10) 0 1 0 1 0						20.402(b)						20.408(e)						80.73(a)(2)(iv)						73.71(b)															
						20.408(a)(1)(i)						80.38(a)(1)						80.73(a)(2)(v)						73.71(e)															
						20.408(a)(1)(ii)						80.38(a)(2)						80.73(a)(2)(vi)						X OTHER (Specify in Abstract below and in Text, NRC Form 385A)															
						20.408(a)(1)(iii)						80.73(a)(2)(i)						80.73(a)(2)(vii)(A)						voluntary report															
						20.408(a)(1)(iv)						80.73(a)(2)(ii)						80.73(a)(2)(vii)(B)																					
20.408(a)(1)(v)						80.73(a)(2)(iii)						80.73(a)(2)(x)																											
LICENSEE CONTACT FOR THIS LER (12)																																							
NAME David W. Rogers; Technical Engineer; Palisades																				TELEPHONE NUMBER 6 1 1 6 7 1 6 4 1 - 8 9 1 3																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																							
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC						CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC									
A			B 1 P F C V			B 2 1 9 5			Y																														
SUPPLEMENTAL REPORT EXPECTED (14)																																							
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO										EXPECTED SUBMISSION DATE (15)									
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																							
<p>On September 3, 1984, at 1702, an auxiliary operator discovered Low Pressure Safety Injection Flow Control Valve CV-3006 [FCV;BP] to be less than fully open. Visual observation of the valve stem indicated that the valve was open approximately 2½ inches of the 4 inch full open stroke. Palisades Technical Specification 3.3.1(h) requires CV-3006 to be open as a condition for reactor criticality. The reactor was critical, in hot standby condition at the time of discovery. The valve was subsequently hand jacked to the full open position.</p> <p>Investigation determined that the valve could not be stroked further than 3½ inches (other than manually) toward the full open position due to misadjusted packing, which occurred when the valve was refurbished during the 1983/1984 refueling outage. The valve was not recognized as being less than full open due to inadequate local position indication on the valve. Subsequently, the full open position has been positively identified.</p> <p>CV-3006 is required to be open in order to pass flow from the low pressure safety injection pumps [P;BP]. Although CV-3006 was not fully open, analysis indicates that the valve could pass 3750 gpm, which is in excess of the 3000 gpm assumed for accident analyses. Consequently, the occurrence has no safety significance, and no threat to public health or safety resulted.</p>																																							
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IE 22 11																																							



**Consumers
Power
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November 30, 1984

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-017, REVISION 1 -
(CV-3006 FOUND NOT FULLY OPEN)

Attached please find Licensee Event Report 84-017, Revision 1 (CV-3006 Found Not Fully Open). The original report was reportable to the NRC per 10 CFR 50.73 but upon analysis was found not to be reportable. This revision 1 to LER 84-017 is a voluntary report and gives the details of the analysis.

Brian D Johnson
Staff Licensing Engineer

CC Director, Office of Nuclear Reactor Regulation
Director, Office of Inspection and Enforcement
NRC Resident Inspector - Palisades

Attachment

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1/1

OC1184-0012A-NL02