

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) 4 4 INDIAN POINT UNIT 2										DOCKET NUMBER (2) 2, 47 0 5 0 0 0 1 1 1										PAGE 1 OF 012																													
TITLE (4) PREMATURE LIFTING OF SECONDARY SAFETY VALVE																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH		DAY		YEAR		YEAR		SEQUENT		AL		NUMBER		REASON		NUMBER		MONTH		DAY		YEAR		FACILITY NAMES										DOCKET NUMBER (3)															
10		16		84		84		01		16		00		11		15		84																0 5 0 0 0 1 1 1															
OPERATING MODE (9)										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50.73 (b) (1) (Select one or more of the following (11))																																							
POWER LEVEL (10) 0100										20.402(a)										20.402(b)										20.73(a)(2)(iv)										73.71(a)									
										20.402(a)(1)(i)										20.402(a)(1)(ii)										20.73(a)(2)(v)										73.71(a)									
										20.402(a)(1)(iii)										20.402(a)(1)(iv)										20.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
										20.402(a)(1)(v)										20.402(a)(1)(vi)										20.73(a)(2)(vii)																			
										20.402(a)(1)(vii)										20.402(a)(1)(viii)										20.73(a)(2)(viii)																			
										20.402(a)(1)(ix)										20.402(a)(1)(x)										20.73(a)(2)(ix)																			
										20.402(a)(1)(xi)										20.402(a)(1)(xii)										20.73(a)(2)(x)																			
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME																				TELEPHONE NUMBER																													
MICHAEL BLATT																				AREA CODE 914 526-5127																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE		SYSTEM		COMPONENT		MANUFAC		TURER		REPORTABLE		TO NRCOS		CAUSE		SYSTEM		COMPONENT		MANUFAC		TURER		REPORTABLE		TO NRCOS																							
A		S B		R V		C 7		1 0		N																																							
SUPPLEMENTAL REPORT EXPECTED (14)																																																	
YES (If you complete EXPECTED SUBMISSION DATE)																				NO																													
EXPECTED SUBMISSION DATE (15)																				MONTH DAY YEAR																													

ABSTRACT (Limit to 1400 words, i.e., space; ordinary 11/24 inch single-space typewritten text) (16)

On Tuesday, October 16, 1984, the reactor was in the process of being taken critical at the conclusion of a maintenance refueling outage. During the course of rod withdrawal, a steam generator delta P signal tripped the reactor and caused a safety injection signal.

The cause of the event was the lifting of a safety relief valve on the secondary side. The reactor coolant system was cooled down about 50F and the secondary side relief valve setpoint readjusted.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/95

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
INDIAN POINT UNIT 2	01500002147	84	-0116	-00012	OF	02

TEXT (if more space is required, use additional NRC Form 256A (11/73))

On October 16, 1984 the reactor was in the hot shutdown mode at the conclusion of a refueling maintenance outage. During the course of rod withdrawal while proceeding to criticality, a steam generator (number 22) delta P signal tripped the reactor and caused a safety injection signal. The primary system was at a temperature of 546°F and 2234 psig. The steam generator pressure under these conditions corresponds to 1005 psig.

The cause of the event was the premature opening of a safety relief valve, number 48B. The nominal setpoint of this valve is 1110 psig. The reactor coolant system was cooled down and the setpoint of valve 48B was found to be below the specified lower value for operation. The setpoint was readjusted and other relief valve setpoints checked.

Since the primary system was at a pressure higher than the safety injection pressure, no injection took place.

John D. O'Toole
Vice President

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November 15, 1984

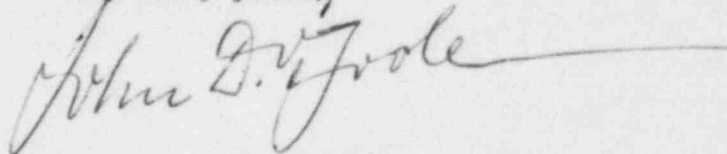
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-84-016-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-84-016-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Dr. Thomas F. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
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King of Prussia, Pa. 19406

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