

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3180-0104
EXPIRES - 8/31/93

FACILITY NAME (1) INDIAN POINT UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 2 4 7 1 OF 0 2										PAGE (3) 1																															
TITLE (4) STEAM GENERATOR LEVEL/FLOW MISMATCH - REACTOR TRIP																																																			
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																																	
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES				DOCKET NUMBER(S)																													
1 0		20		8 4		8 4		0 18		0 0		1 1		19		8 4						0 5 0 0 0 1 1 1																													
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OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § 1.110 (Check one or more of the following) (11)																																																	
		20.402(a)										20.406(a)										<input checked="" type="checkbox"/> 50.73(a)(2)(iv)										73.71(b)																			
POWER LEVEL (10)		0 1 0										20.406(a)(1)(i)										50.36(a)(1)										50.73(a)(2)(iv)										73.71(a)									
												20.406(a)(1)(ii)										50.36(a)(2)										50.73(a)(2)(iv)										OTHER (Specify in Abstract below and in Text, NRC Form 306A)									
												20.406(a)(1)(iii)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(A)																			
												20.406(a)(1)(iv)										50.73(a)(2)(v)										50.73(a)(2)(viii)(B)																			
												20.406(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(vi)																			
LICENSEE CONTACT FOR THIS LER (12)																																																			
NAME MICHAEL BLATT																TELEPHONE NUMBER																																			
																AREA CODE 9 1 4 5 2 6 1 - 5 1 1 2 1 7																																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																			
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																																	
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SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR																													
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)																<input type="checkbox"/> NO																																			
ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-space typewritten lines) (16)																																																			

On October 20, 1984 while at 5% reactor power at the conclusion of a refueling and maintenance outage, a reactor trip occurred as a result of a steam generator low level and steam flow/feedwater flow mismatch signal. The incident occurred after a shift from the Auxiliary Boiler Feed Pumps to the Main Boiler Feed Pumps. The cause of the event is attributed to the operator not adequately responding to plant conditions. There was no effect on plant safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8-31-85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (3)

PAGE (3)

INDIAN POINT UNIT 2

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TEXT (1) more space is provided; use additional NRC Form 2586 (17)

On October 20, 1984 a reactor trip occurred due to a steam generator low level and steam flow/feedwater flow mismatch signal. The plant was at 5% reactor power at the time. The water level in the affected Steam Generator #22 was at 25% of full level.

The plant operator was switching the feedwater supply from Auxiliary Feedwater Pump operation to Main Feedwater Pumps when the level in Steam Generator #22 started to fall. Feedwater supply was established via the electric power Auxiliary Feedwater Pumps. The low flow feedwater regulating valve was actuated to the full open position but did not fully respond. Over the period of time involved the operator had other options to pursue such as reduction in steam demand but did not pursue them. Ultimately a reactor trip occurred because feedwater flow and steam flow became mismatched beyond the trip setpoint. The regulating valve was repaired.

The cause of the event is attributed to the operator not adequately responding to plant conditions. The event was reviewed with the watch personnel involved to assure they understood the conditions that contributed to this event and the actions that would prevent it. There was no effect on plant safety.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

November 19, 1984

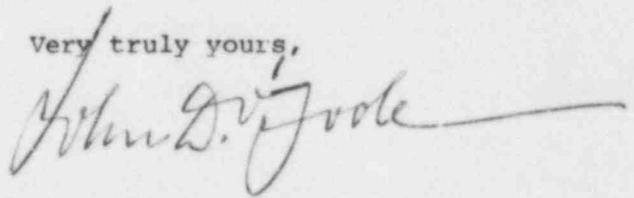
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-84-018-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-84-018-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Dr. Thomas E. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

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