

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)										DOCKET NUMBER (2)				PAGE (3)			
EDWIN I. HATCH, UNIT 2										0 5 0 0 0 3 6 6				1 OF 0 2			

TITLE (4) ESF Actuation on Equipment Room High Temperature

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)													
1	0	3	1	8	4	8	4	0	3	1	0	0	1	1	2	1	8	4	0	5	0	0	0				
																			0	5	0	0	0				

OPERATING MODE (S):		1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)					
POWER LEVEL (10)		190	20.402(b)		20.405(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)	73.71(c)
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(vii)(A)	
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(vii)(B)	
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)									
NAME						TELEPHONE NUMBER			
						AREA CODE			
T. L. Elton, Acting Superintendent of Regulatory Compliance						9 1 2		3 6 7 - 7 8 5 1	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10-31-84 (at approximately 2200 CDT), with the reactor operating at 2430 Mwt (approximately 100% power), the reactor water cleanup (RWCU) inboard isolation valve 2G31-F002 isolated because of high equipment room temperature. No actual or potential safety consequences or implications resulted from this event. The health and safety of the public were not affected by this repetitive event.

This event was caused by a leaking pump shaft seal on RWCU pump 2G31-C001A, and two leaking RWCU pump vent valves.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXP. RES. 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT 2	0 5 0 0 0 3 6 6 8 4 -	0 3 1 -	0 0	0 2	OF	0 2	

TEXT (If more space is required, use additional NRC Form 365A's) (17)

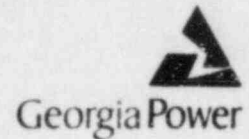
This 30 day LER is required by 10CFR50.73 (a)(2)(iv) because this event shows that an engineered safety feature (ESF) was automatically actuated.

On 10/31/84 (at approximately 2200 CDT) with the reactor operating at 2430 MWT (approximately 100% power), operating personnel determined that reactor water cleanup (RWCU) inboard isolation valve 2G31-F001 (which is an ESF) had isolated as a result of high temperature (approximately 125 degrees F) in the RWCU equipment room. High temperature in the RWCU equipment room was caused by a leaking shaft seal on RWCU pump 2G31-C001A, and two leaking RWCU pump vent valves (2G31-F010A and 2G31-F011A).

RWCU pump 2G31-C001A was tagged out and removed from service per the "EQUIPMENT CLEARANCE AND TAGGING" procedure (HNP-501), thus plant operation continued. This event was the result of equipment failure, and is reportable per 10CFR 50.73 (a)(2)(iv) because an engineered safety feature was actuated (i.e., RWCU inboard isolation valve 2G31-F001 isolated on equipment room high temperature).

Valves 2G31-F010A and 2G31-F011A were repaired and returned to service on 11-08-84. The pump seal on 2G31-C00A was replaced on 11-09-84 and it will be visually examined for leaks during the pump's first startup following repair. No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other system in Unit 2, or Unit 1. The health and safety of the public were not affected by this event. This is a repetitive event as last reported on LER 50-366/1984-07.

Georgia Power Company
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Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444



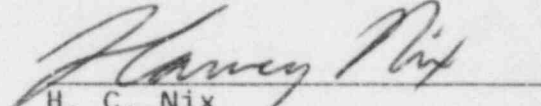
Edwin I. Hatch Nuclear Plant

November 21, 1984
GM-84-1067

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1984-031. This report is required by 10CFR 50.73(a)(2)(iv).


H. C. Nix
General Manager

HCN/TLE/djs

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