

L I C E N S E E F E V E N T R E P O R T ( L E R )

FACILITY NAME (1) Arkansas Nuclear One, Unit Two IDOCKET NUMBER (2) (PAGE (3)  
10151010101 31 61 811CF1011

TITLE (4) Reactor Trip Caused By Dropped Control Element Assembly

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
Month	Day	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
11	01	84	01	01	11	01	84		0151010101

OPERATING MODE (9) 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

POWER LEVEL (10)	11010	20.402(b)	20.405(a)(1)(i)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(ii)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)	
		20.405(a)(1)(iii)	50.36(c)(2)	50.73(a)(2)(vii)	Other (Specify in	
		20.405(a)(1)(iv)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	Abstract below and	
		20.405(a)(1)(v)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	in Text, NRC Form	
			50.73(a)(2)(iii)	50.73(a)(2)(x)	366A)	

Name LICENSEE CONTACT FOR THIS LER (12) Patrick Rogers, Plant Licensing Engineer Telephone Number  
Area  
Code  
5101191641-1311010

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

Cause	System	Component	Manufacturer	Reportable to NPRDS	Cause	System	Component	Manufacturer	Reportable to NPRDS
X	AI A	CI LI	CI 41 91 0	Y					

SUPPLEMENT REPORT EXPECTED (14) EXPECTED SUBMISSION DATE (15) Month Day Year  
1 Yes (If yes, complete Expected Submission Date) X No

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)  
On 10/26/84 at 1137 hours, while in Mode 1 at 100% full power (FP), an automatic reactor trip occurred as a result of a dropped control element assembly (CEA). Penalties generated by the Control Element Assembly Calculators due to the misalignment of CEA #7 and supplied to the Core Protection Calculators resulted in a calculated low DNBR leading to a reactor trip. The Emergency Feedwater System automatically actuated as a result of low steam generator water level. Bypass valve 2CV-0753 for the 'A' main feedwater regulating valve did not close completely but did not adversely affect steam generator water level control. Investigation of the CEA problem revealed that the upper gripper coil for CEA #7 had shorted allowing the rod to drop. The stator coil for CEA #7 was replaced. Resistance readings for the upper gripper coils for all CEA's were taken and found to be acceptable. A failed lift coil was reported in LER 78-022 but a reactor trip did not occur. LER's 84-013 and 84-024 reported reactor trips caused by dropped CEA's but were not the result of coil failures.



ARKANSAS POWER & LIGHT COMPANY

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November 16, 1984

2CAN118404

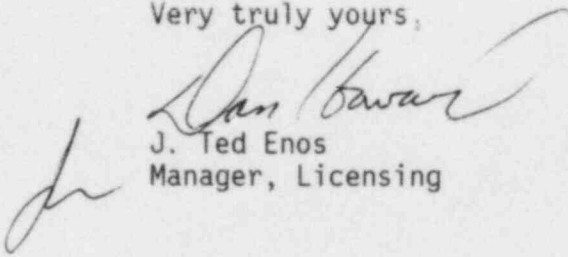
U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2  
Docket No. 50-368  
License No. NPF-6  
Licensee Event Report  
No. 84-026

Gentlemen:

In accordance with 10CFR50.73(a)(2)(iv), attached is the subject report concerning an automatic reactor trip which occurred as a result of a dropped control element assembly.

Very truly yours,

  
J. Ted Enos  
Manager, Licensing

JTE:RJS:lw

Attachment

cc: Mr. Richard C. DeYoung  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. Norman M. Haller, Director  
Office of Management & Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

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