

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Davis-Besse Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 4 6 1					PAGE (3) OF 02								
TITLE (4) Containment Isolation Valve RC229A Exceeded Stroke Time Limit																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)										
0	2	2	0	8	5	8	5	0	0	4	0	0	0	3	2	2	8	5	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																					
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)									
POWER LEVEL (10)		0 9 2				20.405(a)(1)(i)				50.73(a)(2)(v)				73.71(c)									
		20.405(a)(1)(ii)				50.73(a)(2)(vi)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
		20.405(a)(1)(iii)				50.73(a)(2)(viii)(A)																	
		20.405(a)(1)(iv)				50.73(a)(2)(viii)(B)																	
		20.405(a)(1)(v)				50.73(a)(2)(ix)																	
LICENSEE CONTACT FOR THIS LER (12)																							
NAME										TELEPHONE NUMBER													
G. Lammon/E. Caba										AREA CODE		4 1 9 2 5 9 5 0 0 0											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS														
A	C	A	I S V A	4 9 9	Y																		
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Containment Isolation Valve RC229A exceeded the limit of ten seconds per Technical Specification Table 3.6-2 when its closing stroke time measured 10.48 seconds. The required action statement of isolating the penetration within four hours was not met when the operators performing and reviewing ST 5099.08, Miscellaneous Valves Quarterly Test, did not notice the stroke time limit was exceeded. The designated reviewer for the test informed the Operations Supervisor on February 20, 1985, that the limit was exceeded. The action statement of Technical Specification 3.6.3.1 was entered, and the containment penetration was isolated using manual valve RC79. The valve RC229A was repaired on February 21, 1985, and was successfully retested. This removed the plant from the action statement on 1300 hours on February 21, 1985.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Davis-Besse Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 4 6	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	— 0 0 4	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of Occurrence: As part of the Miscellaneous Valves Quarterly Test, ST 5099.08, the quench tank (CA) outlet isolation valve (ISV) RC229A was stroked and its time recorded on the data sheet along with the times of fifty-two other valves. The stroking of this valve occurred on February 7, 1985. The entire test was completed, approved, and submitted to the Surveillance Test Engineer on February 15, 1985. On February 20, 1985, during the trending of the valve stroke times, it was noted that the closing stroke time for RC229A was recorded as 10.48 seconds, while the acceptable limit in Technical Specification Table 3.6-2 is ten seconds. The discrepancy was brought to the attention of the designated reviewer who immediately notified the Operations Supervisor. The valve was retested to check the accuracy of the recorded time since it was less than a half second above the limit. The retest confirmed the valve was exceeding the ten second closing limit, and the action statement of Technical Specification 3.6.3.1 was entered at 1040 hours on February 20, 1985. The penetration was then isolated by closing the manual valve RC79 downstream of RC229A.

The event is being reported under 10CFR50.73(a)(2)(i)(B) as the operation or condition prohibited by the plant's Technical Specifications. Although the data indicated a problem, the condition was not recognized, and the action statement requirements were not met on February 7, 1985, which violated the Technical Specification.

Designation of Apparent Cause of Occurrence: The cause was both personnel error by the operators performing and reviewing the test, and human factors inadequacies regarding the procedure format. Due to the large number of valves being tested, the single acceptance criteria signoff, and the number of people involved in the test, the stroke time for RC229A exceeding the acceptance criteria was not noticed.

Analysis of Occurrence: In each of these penetrations, two containment isolation valves are provided in series. The backup isolation valve RC229B was also tested in Surveillance Test ST 5099.08 on February 7, 1985, and was well within its Technical Specification limits. RC229A did close during the test, but exceeded its limit by 5 percent.

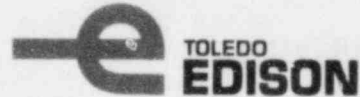
Corrective Action: The individuals involved have been counseled by their supervisor on the importance of verifying acceptance criteria. The circumstances of the event were examined in this discussion and attention to detail was emphasized. Since this particular test is not usually done in its entirety by the same individuals, the procedure, ST 5099.08, will be modified to provide an acceptance criteria verification for each valve rather than a blanket verification for all the valves in order to ensure each valve is properly reviewed.

RC229A was repaired on February 21, 1985, under Maintenance Work Order 1-85-0836-00 and was successfully tested and declared operable. This removed the plant from the action statement at 1300 hours on February 21, 1985.

Failure Data: A previous failure to recognize that all acceptance criteria had not been met was reported in Licensee Event Report NP-33-84-24 (LER 84-22).

Report No: NP-33-85-05

DVR No(s): 85-028



March 22, 1985

Log No. K85-549
File: RR 2 (NP-33-85-05)

Docket No. 50-346
License No. NPF-3

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Gentlemen:

LER No. 85-004
Davis-Besse Nuclear Power Station Unit 1
Date of Occurrence: February 20, 1985

Enclosed is Licensee Event Report 85-004 which is being submitted in accordance with 10CFR50.73, to provide 30 day written notification of the subject occurrence.

Yours truly,

Stephen M. Quennoz
Plant Manager
Davis-Besse Nuclear Power Station

SMQ/ljk

Enclosure

cc: Mr. James G. Keppler,
Regional Administrator,
USNRC Region III

Mr. Walt Rogers
DB-1 NRC Resident Inspector

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