

## LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104  
EXPIRES 9/31/85

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 3										DOCKET NUMBER (2) 01501010278										PAGE (3) 1 OF 013																																
TITLE (4) Mode Switch Mispositioning																																																				
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																						
MONTH			DAY			YEAR			YEAR			SEQUENCE NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAME										DOCKET NUMBER(S)															
01			22			08			58			5			002			000			01			32			18			5													0150101011									
OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																										
POWER LEVEL (10) 01010										20.402(b)										20.406(a)										80.73(a)(2)(iv)										73.71(h)												
										20.406(a)(1)(ii)										80.38(a)(1)										80.73(a)(2)(v)										73.71(a)												
										20.406(a)(1)(iv)										80.38(a)(2)										80.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Test, NRC Form 364A)												
										20.406(a)(1)(iii)										X 80.73(a)(2)(ii)										80.73(a)(2)(vii)(A)																						
										20.406(a)(1)(iv)										80.73(a)(2)(v)										80.73(a)(2)(viii)(B)																						
20.406(a)(1)(v)										80.73(a)(2)(iii)										80.73(a)(2)(ix)																																
LICENSEE CONTACT FOR THIS LER (12)																																																				
NAME J. C. Nagle, Engineer - Special Projects																				TELEPHONE NUMBER AREA CODE 2115 8411-151814																																
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																				
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC				CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																																
SUPPLEMENTAL REPORT EXPECTED (14)																				EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR																						
YES (If you complete EXPECTED SUBMISSION DATE)																				X NO																																

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 3-85-02

On February 20, 1985, with Unit 3 shutdown and the reactor mode switch in the REFUEL position, the automatic start capability of the Residual Heat Removal (RHR) and core spray systems was defeated to permit repairs to the inboard Main Steam Isolation Valves (MSIV's). Technical Specification 3.5.F.3 allows emergency core cooling system inoperability if the reactor mode switch is in the SHUTDOWN position. The cause of the event was failure of a licensed reactor operator to assure correct position of the reactor mode switch as required by procedure. The error was discovered within approximately four hours.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 12/31/89

FACILITY NAME (1)  Peach Bottom Atomic Power Station - Unit 3	DOCKET NUMBER (2)  0 5 0 0 0 2 7 8 8 5 - 0 0 2 - 0 0	LER NUMBER (3)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
					0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of the Event:

On February 20, 1985 at approximately 4:00 a.m. with Unit 3 shutdown and the reactor mode switch in the REFUEL position, the automatic start capability of the Residual Heat Removal (RHR) and core spray systems was defeated to permit repairs to the inboard Main Steam Isolation Valves (MSIV's). Technical Specification 3.5.F.3 allows Emergency Core Cooling System (ECCS) inoperability if the reactor mode switch is in the SHUTDOWN position. The error was discovered within approximately four hours.

Consequences of the Event:

Technical Specification 3.5.F.3 allows low pressure ECCS systems to be inoperable with the mode switch in SHUTDOWN provided that no work is being performed which could potentially drain the vessel. During the short period of time that the mode switch remained in REFUEL, no work was performed which could have potentially drained the vessel. Therefore, the safety consequences of this event are considered minimal.

Cause of the Event:

The cause of the event was failure of a licensed reactor operator to assure that the reactor mode switch was positioned in SHUTDOWN. This requirement is specified as a prerequisite in Special Procedure 779, "Temporary Defeating of RHR and Core Spray Pump Auto Starts During Inboard MSIV Repair", written for this plant configuration.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/86

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
					0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A-2) (17)

Corrective Actions:

The individual responsible for mispositioning the reactor mode switch was counseled on the importance of procedural compliance.

Special Procedure 779 will be modified to preclude future occurrences by the addition of a caution note to better instruct the operators that the mode switch must be in SHUTDOWN while low pressure ECCS is defeated. Additionally, the procedure will be modified to require positive notification to Shift Supervision and the affected reactor operator that low pressure ECCS automatic start capability has been defeated and that the mode switch must remain in SHUTDOWN.

PHILADELPHIA ELECTRIC COMPANY

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PHILADELPHIA, PA. 19101

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March 21, 1985

Docket No. 50-278

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

SUBJECT: Licensee Event Report  
Peach Bottom Atomic Power Station - Unit 3

This LER concerns the improper positioning of the reactor mode switch with ECCS systems defeated.

Reference:	Docket No. 278
Report Number:	3-85-02
Revision Number:	00
Event Date:	February 20, 1985
Report Date:	March 21, 1985
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B).

Very truly yours,



W. T. Ullrich  
Superintendent  
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator  
Region I, USNRC

Mr. T. P. Johnson, Resident Inspector

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