

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Beaver Valley Power Station, Unit 1DOCKET NUMBER (2)
0 5 0 0 0 3 3 4 1 OF 0 1TITLE (4)
Main Steam Safety Valve Lift Settings Outside Allowable Limits

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)							
1	0	1	2	8	4	8	4	0	1	1	0	0	0	0	0	0		
N/A												0	5	0	0	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10)	0, 9, 0	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)						
		20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)						
		20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
		20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
		20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(vii)(B)							
		20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)
NAME: Terry M. McGhee, Acting Technical Advisory Engineer
TELEPHONE NUMBER: 411 264 314 0615

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	SIBR	VI	D	21415	Y				

SUPPLEMENTAL REPORT EXPECTED (14)
☒ YES (If yes, complete EXPECTED SUBMISSION DATE)
NO
EXPECTED SUBMISSION DATE (15)
0 1 1 5 8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/12/84 while in Hot Standby, lift settings on eight (8) of the fifteen (15) main steam safety valves were found outside the Technical Specification 3.7.1.1 operating limits of plus or minus 1%. Of the eight valves, one had a lift setting slightly below the normal operating limit, while seven had lift settings above their normal operating limits. The valves were all adjusted to within the allowable limits and tested satisfactorily. The root cause for the lift settings being outside their allowable limits is under investigation, however, it is believed that setpoint drift is also a contributory factor. The valves are Type 3707 RAX-RT21, manufactured by the Dresser Valve Division. Long-term corrective actions to prevent recurrence are under investigation. The long-term corrective action taken and the root cause determination will be reported when this information is received. Steam generator overpressurization protection was not degraded. The as-found lift settings for the valves designed to provide initial relief were found set within their normal operating limits, with the exception of one valve which was found to be set conservatively low. This is the fourth licensee event report issued for this type of occurrence.

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Duquesne Light

Nuclear Division
P.O. Box 4
Shippingport, PA 15077-0004

Telephone (412) 393-6000

November 11, 1984
ND1SS1:2250

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
LER 84-011

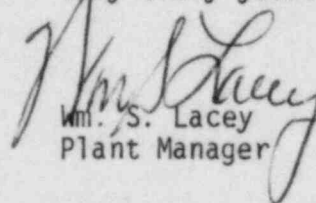
Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region I
Park Avenue
King of Prussia, PA 19046

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 84-011, 10 CFR 50.73.a.2.ii, "Degradation of Safety Valve Operability".

Very truly yours,


Wm. S. Lacey
Plant Manager

md

Attachment

T. E. Murley
November 11, 1983
ND1SS1:2250
Page two

cc: Director of Management & Program Analysis
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Washington, D.C. 20555

C. A. Roteck, Ohio Edison

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