



November 30, 1984  
JPN-84-78

Director of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Attention: Mr. Domenic B. Vassallo, Chief  
Operating Reactors Branch No. 2  
Division of Licensing

Subject: James A. FitzPatrick Nuclear Power Plant  
Docket No. 50-333  
Supplement No. 1 to NUREG-0737  
(Generic Letter No. 82-33)  
Safety Parameter Display System (SPDS)  
Implementation Plan

- References:
1. NRC December 17, 1982 Generic Letter No. 82-33 to All Licensees of Operating Reactors. Includes Supplement No. 1 to NUREG-0737.
  2. PASNY letter, J. P. Bayne to D. B. Vassallo, dated April 15, 1983 (JPN-83-33), regarding the same subject.
  3. NRC letter, D. B. Vassallo to J. P. Bayne, dated June 12, 1984 regarding order confirming licensee commitments on emergency response capability.
  4. NYPA letter, J.P. Bayne to D.B. Vassallo, dated August 24, 1983 (JPN-83-77) regarding Supplement No. 1 requirements for emergency response capability.

Dear Sir:

Generic Letter No. 82-33 (Reference 1) transmitted to the Authority Supplement No. 1 to NUREG-0737 which replaced the corresponding requirements for five NUREG-0737 items. Via Reference 2, the Authority submitted a schedule for completing each of the basic requirements and a description of our plans for the phased implementation and integration of these emergency response capabilities. The NRC subsequently confirmed these commitments with an order dated June 12, 1984 (Reference 3).

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In accordance with the Authority's commitments, attached is a document entitled "Safety Parameter Display System Implementation Plan for the James A. FitzPatrick Nuclear Power Plant." This plan includes a description of the SPDS itself; schedules for the design, development, installation, training and full operation; and the elements of the verification and validation plan. The schedules included in the SPDS Implementation Plan (with the exception of the dates for full SPDS operation and fully functional Technical Support Center) are for information only and not to be construed as formal Authority commitments.

As detailed in Section 4.0 of the attached Implementation Plan, the FitzPatrick SPDS will be fully operational and operators trained at the end of the Reload 8/Cycle 9 refueling outage which is currently scheduled for late 1987 or early 1988. Because the FitzPatrick Technical Support Center (TSC) will not be considered fully functional until an operational SPDS display terminal is installed there, the TSC will also be fully functional at the end of the Reload 8/Cycle 9 refueling outage.

In Reference 4, the Authority committed to complete a new Emergency Operations Facility (EOF) by June 28, 1985. When the Authority made this commitment, it was not our intention to include the installation and operation of an SPDS as the method for providing the required plant parameters. When we described our plans to use the SPDS to communicate the plant conditions to the TSC (Section 6.1.8 of Reference 2), firm schedules for the new EOF and SPDS had not been developed.

If the NRC staff does not agree with the Authority's interpretation (i.e. the June 28, 1985 date for EOF completion excludes installation and operation of equipment to collect, store, analyze, display and communicate information on containment conditions, radiological releases and other EOF functions) then, the Authority hereby requests that the NRC amend their order dated June 12, 1984 (Reference 3) to reflect the scheduled completion date for the FitzPatrick EOF. Because the new dose assessment/meteorological computer system is also scheduled for completion June 28, 1985, no change to the June 12, 1985 order is required to accomodate its schedule.

In our April 15, 1983 letter (Reference 2), the Authority requested that the NRC perform a preimplementation review of the SPDS Verification and Validation Program Plan. The Authority hereby withdraws this request - no preimplementation review is expected.

We are proceeding with the implementation of this system in accordance with schedules transmitted separately. Your prompt review of this implementation plan and the associated safety analysis will be appreciated. Based upon discussions with NRC staff members at public meetings, we expect that a letter regarding the acceptability of our plans will be issued within two months.

If you have any questions about this plan, please contact  
Mr. J. A. Gray, Jr. of my staff.

Very truly yours,

*C. A. McNeill, Jr.*

*for* C. A. McNeill, Jr.  
Senior Vice President  
Nuclear Generation

cc: Office of the Resident Inspector  
U. S. Nuclear Regulatory Commission  
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