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D. O. Foster
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November 30, 1984

Assistant Director for Licensing
Attention: Ms. Elinor G. Adensam, Chief
Licensing Branch #4
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

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NRC DOCKET NUMBERS 50-424 AND 50-425
CONSTRUCTION PERMIT NUMBERS CPPR-108 AND CPPR-109
VOGTLE ELECTRIC GENERATING PLANT - UNITS 1 AND 2
DRAFT SER OPEN ITEM RESPONSE SCHEDULE

REFERENCE: Letter dated 11/6/84, T. M. Novak to D. O. Foster

Dear Mr. Novak:

Your letter, referenced above, requested that a proposed schedule for written responses to the open items identified in the Draft SER be provided. This schedule is attached. Please note that certain items have been identified as closed. It is my understanding that those items, which we have identified as closed, will be discussed in a meeting with your staff on December 13, 1984 to verify the item status.

If you have any questions concerning the attached information, do not hesitate to contact me.

Yours truly,

D. O. Foster
D. O. Foster

DOF/JAB/sw
Attachment

xc: M. A. Miller
R. A. Thomas
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Listing of open items

Item	PROPOSED RESPONSE DATE
(1) Design basis temperatures for auxiliary systems and components	1/31/85
(2) Upgrade of operational meteorological measurements program	2/28/85
(3) Foundation competency of clay marl stratum	2/28/85
(4) Verification FSAR commitments on compaction of Category 1 backfill	2/28/85
(5) Submittal and evaluation of settlement records	2/28/85
(6) Foundation design and construction information on radwaste buildings and tunnels	1/31/85
(7) Bearing capacity stability	12/21/84
(8) Long-term groundwater and settlement monitoring requirements	1/31/85
(9) Acceptability of variations in soil dynamic properties	2/28/85
(10) Final pipe whip and jet impingement evaluation for high-energy piping.	3/30/85
(11) Clarification of pipe break criteria and pipe whip restraints	12/21/84
(12) Methods to relate measured vibration values to stress levels	12/21/84
(13) Safety-related instrument lines in vibration monitoring program	12/21/84
(14) Design of seismic interface anchors	12/21/84
(15) Use of damping values and equivalent static factors	12/21/84
(16) Piping analysis procedures for main steam and feedwater piping outside containment	12/21/84

Item	
(17) Methodology for load combinations	12/21/84
(18) Piping for service level C and D loadings	12/21/84
(19) Compliance with NUREG-0737, Item II.D.1	12/21/84
(20) Design of safety and relief valves	12/21/84
(21) Design and construction of component supports	12/21/84
(22) Snubber pre-service examination and preoperational testing program	12/21/84
(23) Preservice and inservice testing of pumps and valves	12/21/84
(24) Acceptable leak rates	12/21/84
(25) Seismic and dynamic equipment qualification	1/31/85 (EQDP completed 4/85)
(26) Pump and valve operability assurance	1/31/85
(a) Extent to which standards are used	
(b) Compliance with RG 1.148	
(c) Methods and standards for qualification	
(d) Qualification of pump and motor	
(e) Aging and sequence of environmental conditions in maintenance program	
(f) Pumps affected by static shaft analysis	
(g) Generic testing criteria for qualifying check valves	
(h) Administrative control of component qualification	
(i) Onsite audit	
(j) Dependability of containment isolation (purge valves)	
(k) Long-term operability of deep draft pumps (IE Bulletin 79-15)	

Item	
(27)	Sensitivity of CVCS letdown monitor for detecting fuel rod failures 1/31/85
(28)	Postirradiation fuel surveillance program additional surveillance 2/28/85
(29)	Postirradiation fuel surveillance program disposition of failed fuel 2/28/85
(30)	Flow measurement capability with crud buildup 2/28/84
(31)	Thermal-hydraulic design comparison 12/21/84
(32)	Loose parts monitoring system Closed; see A-10, Q 492.1
(33)	Compliance with NUREG-0737, Item II.F.2 2/28/85
(34)	Steamline break DNBR 1/31/85
(35)	Overpressure protection during low temperature operation TBD
(36)	Preservice inspection program 1/31/85
(37)	Impact test data and C_v curve for vessel beltline materials 1/31/85
(38)	Steam generator tube preservice inspection 1/31/85
(39)	Effect of neutron irradiation damage on limiting weld metal TBD
(40)	Withdrawal schedules for surveillance specimens 1/31/85
(41)	Pressure-temperature curves to include closure flange regions 2/28/85
(42)	Natural circulation boration and cooldown tests 2/28/85
(43)	RHRS operation above 450 psig Closed; see A-10, Q 440.68
(44)	Target Rock valves in RVHVS 1/31/85
(45)	Compliance with NUREG-0737, Item II.K.1.5 1/31/85
(46)	Compliance with NUREG-0737, Item II.K.3.10 1/31/85

Item

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| (47) | Operator errors during switchover to recirculation | Closed; see A-10, Q 440.64 |
| (48) | Analysis for large break LOCA with $C_D = 1.0$ | Closed; see A-11, Q 440.134 |
| (49) | Air leakage discrepancy in control room leak rate | 1/31/85 |
| (50) | Data used to estimate control room dose following a LOCA | 1/31/85 |
| (51) | Toxic gas evaluation of chemicals | 2/28/85 |
| (52) | Compliance with RG 1.52 and ANSI 509 | 1/31/85 |
| | (a) No high alarm and trip-alarm signals provided in control room for a temperature sensor located between the heater and first HEPA filter | |
| | (b) No recorded indication is provided in the control room for the pressure drop across the first HEPA filter | |
| (53) | Justification for not providing a cooling mechanism for the ESF filtration units | 1/31/85 |
| (54) | Design modification for automatic reactor trip using shunt coil trip attachment | 1/31/85 |
| (55) | Level measurement errors resulting from environmental temperature effects on level instrument reference legs | 2/28/85 |
| (56) | Auxiliary feedwater system | 1/31/85 |
| (57) | Override of isolation signals | Closed; see A-11, Q 420.29, Q420.30 and Q420.41 |
| (58) | Isolators used in the BOP design | Closed; see A-11, Q 420.10 |
| (59) | Auxiliary relays used with no-go tested slave relays | Closed; see A-11, Q 420.15 and Q 420.36 |
| (60) | Electrical tunnel ventilation system | 1/31/85 |
| (61) | Control room ventilation isolation | 1/31/85 |
| (62) | Emergency response capability - RG. 1.97, Rev. 2 | Under NRC review |

Item		
(63)	Compliance with NUREG-0737, Item II.D.3	Closed; see A-11, Q 420.8
(64)	Bypass and inoperable status panel- conformance to Position C.2 of RG 1.47	1/31/85
(65)	IE Bulletin 79-27, Loss of non-class 1E instrumentation and control power system bus during operation	Closed; see A-11, Q 420.2
(66)	Freeze protection for instrumentation sensing and sampling lines	Closed; see A-11, Q 420.11
(67)	RCS overpressure protection during low temperature operation	Closed; see A-11, Fig. 7.2.1-1
(68)	Compliance with NUREG-0737, Item II.K.3.1	Closed; see A-11, Q 420.55
(69)	Instrumentation for process measurements used for safety functions	1/31/85
(70)	High-energy-line breaks and consequential control system failures	1/31/85
(71)	Control system failure caused by malfunctions of common power source or instrument line	Closed; see A-11, Q 420.6
(72)	Compliance with NUREG-0737, Item II.B.3	2/28/85
	(a) Criterion 2	
	(b) Criterion 10	
(73)	Fire hazards analysis	2/28/85
(74)	Fire doors	2/28/85
(75)	Fire dampers	2/28/85
(76)	Soundproofing materials	2/28/85
(77)	Safe shutdown	2/28/85
(78)	Alternate shutdown	2/28/85
(79)	Power supplies for ventilation	2/28/85

Item	
(80) Fire detection	2/28/85
(81) Valve supervision	2/28/85
(82) Automatic sprinkler systems	2/28/85
(83) Standpipes	2/28/85
(84) Halon 1301 systems	2/28/85
(85) Control room complex	2/28/85
(86) Secondary water chemistry monitoring and control program	2/28/85
(a) Sampling schedule and control limits	
(b) Procedures	
(c) Sampling points	
(d) Data management	
(e) Responsible authority	
(87) Quality assurance for main condenser evacuation system	2/28/85
(88) Quality assurance for turbine gland sealing system	2/28/85
(89) Volume reduction system	Under NRC review
(90) Initial training program	2/28/85
(a) Simulator training	
(b) Walkthrough training	
(c) Review and audit	
(d) Description of SRO training	
(e) Description of training program for heat transfer, fluid flow and thermodynamics	
(f) Training complete before preop tests begin	
(g) Numbers of personnel for whom training and licensing is planned to meet Tech Specs	

Item		
(91)	Licensed operator requalification training program	2/28/85
	(a) Implementation schedule	
	(b) Procedures for record retention	
	(c) Lack of heat transfer, fluid flow and thermodynamic training	
	(d) Training for mitigating core damage	
	(e) Review of abnormal and emergency procedures	
(92)	Compliance with NUREG-0737, Item I.A.2.1	2/28/85
	(a) Lack of heat transfer, fluid flow, and thermodynamic training	
	(b) Retesting of simulator response	
(93)	Compliance with NUREG-0737, Item I.A.2.3	2/28/85
(94)	Compliance with NUREG-0737, Item II.B.4	2/28/85
(95)	Training for nonlicensed plant staff	2/28/85
	(a) Organization teaching the course	
	(b) Distribution of training	
	(c) Health physics training for mechanical and electrical maintenance personnel	
	(d) Refresher instruction	
	(e) Schedule	
	(f) Number of personnel for whom training is planned to meet Tech Specs	
(96)	Fire protection training	2/28/85
	(a) Fire fighting plan	
	(b) Content of instruction as per BTP CMEB 9.5-1 (e.g., meetings every 3 months, drills, refresher training)	

Item		
(96)	Fire protection training (continued)	2/28/85
	(c) Qualified individuals as instructors	
	(d) Training for other plant employees	
(97)	Shift technical advisor training	2/28/85
	(a) Mitigating core damage	
	(b) INPO recommendations	
(98)	Procedures generation package	1/31/85
	(a) Plant-specific technical guideline	
	(b) Writer's guide	
	(c) Validation and verification programs	
	(d) Training program description	
(99)	Initial test program	2/28/85
(100)	Technical Specifications to require four valves to be closed during refueling	2/28/85
(101)	Inadvertent boron dilution during modes 3, 4, and 5	1/31/85
(102)	Compliance with NUREG-0737, Items II.K.3.1/II.K.3.2	2/28/85
(103)	Radiological consequences of SGTR	2/28/85
(104)	Operator action in event of steam generator tube rupture	2/28/85
(105)	Compliance with regulatory guides	2/28/85
(106)	Operational QA program	2/28/85
(107)	Detailed control room design review	3/1/86
(108)	Safety parameter display system	9/1/85