

## LICENSEE EVENT REPORT (LER)

|   |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |                              |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
|---|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|------------------------------|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|--|----------|--|--|--|--|--|--|--|--|--|---|--|--|--|--|--|--|--|--|--|---|--|--|--|--|--|--|--|--|--|
| FACILITY NAME (1)<br>Washington Nuclear Plant - Unit 2                                  |  |  |  |  |  |  |  |  |  | DOCKET NUMBER (2)<br>0 5 0 0 0 3 9 7   |  |  |  |  |  |  |  |  |  | PAGE (3)<br>1 OF 0 3         |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| TITLE (4)<br>Incorrect Fire Detector Installation                                       |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |                              |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| EVENT DATE (5)<br>0 2 7 3 8 5 8 5   |  |  |  |  |  |  |  |  |  | LER NUMBER (6)<br>0 1 5 0 0 0  |  |  |  |  |  |  |  |  |  | REPORT DATE (7)<br>3 1 4 8 5 |  |  |  |  |  |  |  |  |  | OTHER FACILITIES INVOLVED (8)<br>FACILITY NAMES<br>DOCKET NUMBER(S)<br>0 5 0 0 0 |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| OPERATING MODE (9)<br>1   |  |  |  |  |  |  |  |  |  | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11) |  |  |  |  |  |  |  |  |  | 20.402(b)                    |  |  |  |  |  |  |  |  |  | 20.406(e)  |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(iv)  |  |  |  |  |  |  |  |  |  | 73.71(b) |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| POWER LEVEL (10)<br>1 0 1 0   |  |  |  |  |  |  |  |  |  | 20.406(a)(1)(i)  |  |  |  |  |  |  |  |  |  | 50.36(e)(1)                  |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(v)   |  |  |  |  |  |  |  |  |  | 73.71(c)   |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
|   |  |  |  |  |  |  |  |  |  | 20.406(a)(1)(ii)   |  |  |  |  |  |  |  |  |  | 50.36(e)(2)                  |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(vi)  |  |  |  |  |  |  |  |  |  | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
|   |  |  |  |  |  |  |  |  |  | 20.406(a)(1)(iii)  |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(i)               |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(vii)(A)  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
|   |  |  |  |  |  |  |  |  |  | 20.406(a)(1)(iv)   |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(ii)              |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(vii)(B)  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
|   |  |  |  |  |  |  |  |  |  | 20.406(a)(1)(v)  |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(iii)             |  |  |  |  |  |  |  |  |  | 50.73(a)(2)(ix)  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| LICENSEE CONTACT FOR THIS LER (12)<br>NAME<br>R. L. Koenigs, Compliance Engineer        |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |                              |  |  |  |  |  |  |  |  |  | TELEPHONE NUMBER<br>AREA CODE<br>5 1 0 9 3 1 7 7 - 1 2 5 0 1 1                   |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2279    |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |                              |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| CAUSE   |  |  |  |  |  |  |  |  |  | SYSTEM   |  |  |  |  |  |  |  |  |  | COMPONENT                    |  |  |  |  |  |  |  |  |  | MANUFACTURER   |  |  |  |  |  |  |  |  |  | REPORTABLE TO NRC  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |
| A   |  |  |  |  |  |  |  |  |  | TLC  |  |  |  |  |  |  |  |  |  | -                            |  |  |  |  |  |  |  |  |  | -  |  |  |  |  |  |  |  |  |  | -  |  |  |  |  |  |  |  |  |  | -        |  |  |  |  |  |  |  |  |  | - |  |  |  |  |  |  |  |  |  | N |  |  |  |  |  |  |  |  |  |
| SUPPLEMENTAL REPORT EXPECTED (14)<br>YES (If yes, complete EXPECTED SUBMISSION DATE) NO |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |                              |  |  |  |  |  |  |  |  |  | EXPECTED SUBMISSION DATE (15)<br>MONTH DAY YEAR                                  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |          |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |   |  |  |  |  |  |  |  |  |  |

ABSTRACT (Limit to 1400 spaces - e. space - s. y. - from single-space typewritten lines) (16)

During the implementation of a Plant modification, the fire detection instrument for the Residual Heat Removal (RHR) Valve Room was erroneously removed. Thus from 9/9/84 until 2/13/85, when this condition was discovered, the RHR valve room had no installed fire detection instrument and was not monitored by a fire watch patrol. This does not meet Plant Technical Specification requirements.

8503260214 850314  
PDR ADOCK 05000397  
S PDRIF 22  
1/1

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

| FACILITY NAME (1)                 | DOCKET NUMBER (2)   | LER NUMBER (6) |                   |                 | PAGE (3) |     |        |
|-----------------------------------|---------------------|----------------|-------------------|-----------------|----------|-----|--------|
|                                   |                     | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER |          |     |        |
| Washington Nuclear Plant - Unit 2 | 0 5 0 0 0 3 9 7 8 5 | —              | 0 1 5             | —               | 0 0      | 0 2 | OF 0 3 |

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

- a) Power Level - 100%
- b) Plant Mode - 1

Event

On 9/9/84 a Maintenance Work Request (MWR) directing a change in location for Fire Protection (FP) detector number 16/14 was implemented. The craftsmen performing the work removed and relocated FP detector 16/12 in error.

Attached to the MWR, in addition to the Plant Modification Record (PMR) specifying the work which was to be performed, was an information only copy of a previously implemented Plant Engineering Directive (PED) which had directed the relocation of FP detector 16/12. The MWR instructions, which referred to both engineering directives, and the attached previously completed "information only" PED were intended to provide additional information to the craftsman, but instead created some confusion, resulting in detector 16/12 being moved to its previous location.

FP detector 16/12 provides fire indication to the control room for the RHR valve room on the 522' elevation in the Reactor Building. Detector 16/12 was removed on 9/9/84 and went undetected until 2/13/85 when, during the performance of a Technical Specification surveillance test, it was discovered that detector 16/12 was not installed at the correct location in the RHR valve room.

Immediate Corrective Action

The RHR valve room was added to the Plant Fire Watch tour.

Further Corrective Action

- o A Maintenance Work Request (MWR) has been generated to move the detectors to their correct locations in accordance with the requirements of both Plant modification documents.
- o The lack of adequate detector identification markings in the field may have contributed to this event. An MWR has been initiated to physically identify the detectors in the field.
- o Plant supervision will instruct system and maintenance engineers to be precise when preparing MWR work instructions, to only include information needed to complete the task and to become more involved in the implementation of MWR's dealing with changes to Plant configuration.
- o The Plant Maintenance Manager will reemphasize to all Maintenance personnel, the importance of clearly understanding work instructions prior to taking action. When clarifications or further understanding is considered necessary, Maintenance personnel will be encouraged to request assistance.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

| FACILITY NAME (1)                 | DOCKET NUMBER (2) | LER NUMBER (6) |                   |                 | PAGE (3) |     |        |
|-----------------------------------|-------------------|----------------|-------------------|-----------------|----------|-----|--------|
|                                   |                   | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER |          |     |        |
| Washington Nuclear Plant - Unit 2 | 0500039785        | —              | 0115              | —               | 010      | 013 | OF 013 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Safety Significance

The area monitored by this detector contains only one valve in the RHR system (RHR-V-42B) and affected only the Loop B RHR low pressure coolant injection line. There were no fires in this area, no flammable material other than the valve, and the valve was operable during the entire period. The longest this condition could have existed undetected was approximately 6 months due to the Technical Specification surveillance test requirements.

Similar Events

None

## Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

March 14, 1985

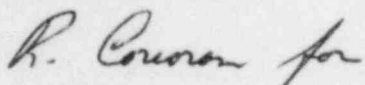
Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2  
LICENSEE EVENT REPORT NO. 85-015

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-015 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,



J. D. Martin (M/D 927M)  
WNP-2 Plant Manager

JDM:mm

Enclosure:

Licensee Event Report No. 85-015

cc: Mr. John B. Martin, NRC - Region V  
Mr. A. D. Toth, NRC - Site (901A)  
Ms. Dottie Sherman, ANI  
INPO Records Center - Atlanta, GA

IE22  
1/1