

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 1 7										PAGE (3) 1 OF 0 1 2									
TITLE (4) Temperature Monitoring Deficiency																													
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)															
0	2	1	6	8	5	8	5	0	1	9	0	0	0	3	1	4	8	5	0	5	0	0	0						
OPERATING MODE (9) 4			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following): (11)																										
POWER LEVEL (10) 0 0 1 0		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)															
		20.406(a)(1)(i)				50.36(e)(1)				50.73(a)(2)(v)				73.71(c)															
		20.406(a)(1)(ii)				50.36(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)															
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)																			
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)																			
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)																			
LICENSEE CONTACT FOR THIS LER (12)																													
NAME R. L. Koenigs, Compliance Engineer															TELEPHONE NUMBER 510 931 771-1 25101														
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															Ext. 2279														
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	-	-	-	-	-	-	-	-	N																				
SUPPLEMENTAL REPORT EXPECTED (14)															EXPECTED SUBMISSION DATE (15)										MONTH	DAY	YEAR		
YES (If yes, complete EXPECTED SUBMISSION DATE)															X NO														

A recent audit of an Operations surveillance procedure performed each shift revealed an error regarding transfer of information from a Procedure Deviation to the correct procedure step. The Plant Procedure Deviation identified the requirement to monitor Residual Heat Removal (RHR) pump room area temperature "at all times". Technical Specification 3/4.7.8 applicability includes the requirement to monitor these area temperatures whenever equipment within that specified area is required to be operable. On 2/16/85, temperature monitoring of the RHR pump room areas was not accomplished in accordance with Plant Technical Specifications.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0500039785	-07	9	-000	2	OF	02

TEXT (If more space is required, use additional NRC Form 386A's) (17)

Plant Conditions

- a) Power Level - 0%  
b) Plant Mode - 4

Event

A Plant Procedure Deviation specified that various area temperatures were to be monitored once per shift "at all times". This involved areas within the RHR pump rooms A and B. The affected indicators were LD-TS-608 A,B,C and D.

Technical Specification 3/4.7.8 requires these indicators be monitored at least once per 12 hours whenever equipment in the area is required to be operable. Instructions were incorrectly transferred from the Plant Procedure Deviation to the actual procedure that was used on 2/16/85. The note "at all times" was inadvertently added to step #37 rather than step #36 of the procedure. Due to the incorrect transfer of instructions, temperature monitoring of the RHR pump areas was not accomplished in accordance with Plant Technical Specifications.

Immediate Corrective Action

None

Further Corrective Action

The Plant Surveillance Procedure was revised and received Plant Operations Committee approval on 3/13/85. This revision incorporated all outstanding procedure deviations to the surveillance procedure.

Safety Significance

The purpose for monitoring these temperatures is to ensure that safety-related equipment does not become exposed to temperatures in excess of its environmental qualifications. The required temperature monitoring was accomplished both the day before and day after the event date with no high temperature conditions noted in these areas. Modes 4 and 5 require that the Reactor be shutdown with an average coolant temperature of less than 200°F. Review of Leak Detection (LD) system temperature recordings for these areas revealed that at no time during this event (mode 4 condition) were temperatures encountered which would have compromised equipment qualification. The maximum temperature recorded by LD instrumentation in these RHR pump rooms was approximately 75°F.

This event is judged to be of no safety significance.

Similar Events

LER 84-126

## Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

March 14, 1985

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2  
LICENSEE EVENT REPORT NO. 85-019

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-019 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,

J. D. Martin (M/D 927M)  
WNP-2 Plant Manager

JDM:mmm

Enclosure:  
Licensee Event Report No. 85-019

cc: Mr. John B. Martin, NRC - Region V  
Mr. A. D. Toth, NRC - Site (901A)  
Ms. Dottie Sherman, ANI  
INPO Records Center - Atlanta, GA

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