NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	1 OF 0 2

TITLE (4)
Containment Overtemperature Condition

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
											0 5 0 0 0
0 2	2 4	8 5	8 5	-	0 1 8	- 0 0	0 3	1 4	8 5		0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	Special Report Tech. Spec. 3.7.8.a
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
R. L. Koenigs, Compliance Engineer	5 0 9 3 7 7 - 1 2 5 0 1 1 Ext. 2279

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	V B	- - -	- - -	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 2/24/85, with the Plant at 100% power, a Containment Monitoring System (CMS) temperature indicator exceeded 150°F for greater than 8 hours. This does not meet the requirements of Plant Technical Specification (T.S.) 3/4.7.8.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Conditions

a) Power Level - 100%
b) Plant Mode - 1

Event

At 1000 hours on 2/24/85, CMS-TI-51 readings exceeded the T.S. 3/4.7.8 limit (< 150°F). The Plant was operating at 100% power and Circulating Water System cooling towers were alternately being taken out of service for electrical repairs. This caused higher Plant Service Water (TSW) temperatures and resulted in higher Reactor Closed Cooling (RCC) system water temperatures. Since RCC supplies water to the cooling coils of the containment cooling fans, drywell temperatures are a direct function of RCC water temperature. Thus, when RCC water temperature increased due to rising outside air temperatures and cooling tower maintenance, drywell temperature (as monitored by CMS-TI-51) rose to 150°F. The maximum temperature recorded was 156°F and at 2200 hours on 2/24/85, CMS-TI-55 temperature was below 150°F.

Immediate Corrective Action

Containment temperatures were continuously monitored to ensure compliance with T.S. average and maximum temperature limits. The maximum temperature was less than 180°F and the average containment temperature remained below 135°F (both values are within acceptable T.S. values).

Further Corrective Actions

- Cooling tower repairs were completed resulting in lower RCC water temperatures and reduced containment drywell temperatures. All monitored temperatures returned to within T.S. allowable values.

- A Plant Modification Record (PMR) has been initiated to provide more constant TSW temperatures during summer conditions. This PMR is currently in the design stage.

Safety Significance

The time and magnitude of this overtemperature condition are judged to have a negligible short term effect when compared to the temperature aging effects experienced over a normal Plant operating lifetime. This localized overtemperature condition in an area above the sacrificial shield wall which contains a limited number of safety-related components, has not jepordized the health and safety of the public or plant personnel.

Similar Events

LER 84-034 Revision 0 through 4

Washington Public Power Supply System
P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

March 14, 1985

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-018

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-018 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,

R. Converse for

J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 85-018

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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