

PUBLIC SERVICE COMPANY OF COLORADO
FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT

NO. 133

February, 1985

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FORM 209 2-2 0218

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PDR ADOCK 05000267
R PDR

This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License DPR-34. This report is for the month of February, 1985.

1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

The reactor remained shutdown during the entire month of February while investigation, refurbishment, and testing of the control rod drives continued.

"1A" helium circulator is being refurbished and analyzed in San Diego. A draft report, "Evaluation of Fort St. Vrain Metallic Components Exposed to Primary Coolant Chloride Contamination" is being reviewed and finalized for submittal to the Nuclear Regulatory Commission in March.

Extensive preparations to ready the fuel deck for control rod drive and orifice assembly (CRDOA) refurbishment and cable replacement have been completed. Design of the refurbishment equipment necessary for working on up to five assemblies simultaneously is complete, and functional. Contract personnel arrived, went through necessary training, and began work on the first control rod drive and orifice assembly on February 6, 1985. Work is proceeding much slower than expected due to procedural problems and parts availability. One control rod drive orifice and assembly has been fully refurbished and placed back in the core. The "assembly line" phase of refurbishment began on February 22, 1985, and four control rod drive orifice assemblies are presently in various stages of refurbishment.

2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATIONS

None

4.0 MONTHLY OPERATING DATA REPORT

Attached

OPERATING DATA REPORT

DOCKET NO. 50-267

DATE March 11, 1985

COMPLETED BY Frank Novachek

TELEPHONE (303) 785-2224

OPERATING STATUS

NOTES

1. Unit Name: Fort St. Vrain
2. Reporting Period: 850201 through 850228
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

9. Power Level To Which Restricted, If Any (Net MWe): 280
10. Reasons for Restrictions, If Any: Per commitment to the NRC, long term operation above 85% power is pending completion of B-0 startup testing.

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	672	1,416	49,681
12. Number of Hours Reactor Was Critical	0.0	0.0	27,151.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	0.0	0.0	18,468.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	9,861,714.4
17. Gross Electrical Energy Generated (MWH)	0	0	3,248,594
18. Net Electrical Energy Generated (MWH)	-1,878	-3,895	2,924,063
19. Unit Service Factor	0.0	0.0	37.2
20. Unit Availability Factor	0.0	0.0	37.2
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	17.8
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	17.8
23. Unit Forced Outage Rate	100.0	100.0	48.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 850301 through 850415, 1104 hours, Control Rod Drive Refurbishment.

25. If Shut Down at End of Report Period, Estimated Date of Startup: 850416

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

Forecast

N/A

N/A

N/A

Achieved

N/A

N/A

N/A

AVERAGE DAILY UNIT POWER LEVEL

TSP-3
Attachment-3A
Issue 2
Page 1 of 1

Docket No. 50-267

Unit Fort St. Vrain #1

Date March 11, 1985

Completed By Frank Novachek

Telephone (303) 785-2224

Month February, 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0.0</u>
2	<u>0.0</u>
3	<u>0.0</u>
4	<u>0.0</u>
5	<u>0.0</u>
6	<u>0.0</u>
7	<u>0.0</u>
8	<u>0.0</u>
9	<u>0.0</u>
10	<u>0.0</u>
11	<u>0.0</u>
12	<u>0.0</u>
13	<u>0.0</u>
14	<u>0.0</u>
15	<u>0.0</u>
16	<u>0.0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0.0</u>
18	<u>0.0</u>
19	<u>0.0</u>
20	<u>0.0</u>
21	<u>0.0</u>
22	<u>0.0</u>
23	<u>0.0</u>
24	<u>0.0</u>
25	<u>0.0</u>
26	<u>0.0</u>
27	<u>0.0</u>
28	<u>0.0</u>
29	<u>N/A</u>
30	<u>N/A</u>
31	<u>N/A</u>

*Generator on line but no net generation.

50-267

Fort St. Vrain #1

March 11, 1985

Frank Novachek

785-2224

UNIT NAME

DATE

COMPLETED BY

TELEPHONE

REPORT MONTH February, 1985

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
84-006	850201	F	672	A	3	50-267/84-008	AA	JC	Control Rod Drive Refurbishment

REFUELING INFORMATION

1. Name of Facility	Fort St. Vrain Unit No. 1
2. Scheduled date for next refueling shutdown.	4th Refueling: February 1, 1986
3. Scheduled date for restart following refueling.	May 1, 1986
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	No
If answer is yes, what, in general, will these be?	-----
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10 CFR Section 50.59)?	No
If no such review has taken place, when is it scheduled?	1985
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	-----
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	-----
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	a) 1482 HTGR fuel elements b) 143 spent fuel elements

REFUELING INFORMATION (CONTINUED)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

- * The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accomodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.



Public Service Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651

March 15, 1985
Fort St. Vrain
Unit #1
P-85078

Office Of Inspection and Enforcement
ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket No. 50-267

REFERENCE: Facility Operating
License No. DPR-34

SUBJECT: Monthly Operations
Report-February, 1985

Dear Sir:

Enclosed, please find our Monthly Operations Report for the month of February, 1985.

Sincerely,

J. W. Gahm
Manager, Nuclear Production

Enclosure

cc: Mr. John T. Collins

JWG/djm

IERA
1/1
Original
To: Region 4