

LICENSEE EVENT REPORT (LER)

Supplemental Report: Date of previous report 1/16/85

FACILITY NAME (1) Dresden Nuclear Power Station										DOCKET NUMBER (2) 0 5 0 0 0 2 3 7 1										PAGE (3) 1 OF 0 2									
TITLE (4) 4 KV Feed Breaker Failure																													
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES N/A					DOCKET NUMBER(S) 0 5 0 0 0															
1	2	2	0	8	4	8	4	0	2	5	0	1	0	2	2	1	8	5	N/A					0 5 0 0 0					
OPERATING MODE (9)		N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																									
POWER LEVEL (10)		0 0 10		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)													
				20.405(a)(1)(i)				50.36(a)(1)				X 50.73(a)(2)(v)				73.71(c)													
				20.405(a)(1)(ii)				50.36(a)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 356A)													
				20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)																	
				20.405(a)(1)(iv)				X 50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)																	
				20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)																	
LICENSEE CONTACT FOR THIS LER (12)																		TELEPHONE NUMBER											
NAME Ronald Jackson (X-549)																		AREA CODE 8 1 5 9 4 2 - 2 9 2 0											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																													
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC										
X	EIB	1 15 12	G 10 18 10	Y																									
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR											
YES (If yes, complete EXPECTED SUBMISSION DATE)														X NO															

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During relay surveillance test while in refuel mode, the 4 KV main feed breaker 152-2430 on bus 24-1 failed to trip open. Safety significance was minimal since the unit was in a refueling outage and the redundant breaker on bus 23-1 did not exhibit any problems during the relay surveillance tests.

A subsequent failure to trip occurred when Operational Analysis Department personnel tried to trip the breaker with the local control switch. However, the breaker did trip mechanically when the local trip button was depressed.

The cause of failure was attributed to dirty auxiliary breaker contacts in the trip control logic. The contacts were cleaned and the breaker returned to service after successful testing.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	— 0 2 5	— 0 1	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

During relay surveillance test while in refuel mode, the main feed breaker 152-2430 on bus 24-1 failed to trip open. Failure to trip also occurred when the local control switch was utilized to trip the breaker. However, the breaker did trip mechanically upon depression of the local trip button. All other similar 4 KV breakers were tested with the exception of seven and no problems were noticed. The Operational Analysis Department (OAD) plans to test the remaining seven breakers prior to unit startup from the current refueling outage. All unit 4 KV breakers are normally trip tested during every refuel outage.

Following initial inspection, the trip coil was found burned and replaced but the breaker did not successfully trip when tested. Further inspection revealed dirty auxiliary breaker contacts as being the cause of the breaker's failure. A high resistance path was generated by the dirty contacts which resulted in a voltage drop across them, thus an insufficient voltage was available to fully energize the coil. However, this applied voltage was enough to burn out the trip coil since it was inadvertently applied continuously for approximately 4 minutes, thereby exceeding the coil's rating. The continuous application occurred because the OAD Test Engineer did not reset the release handle on the lock-out relay while trying to determine why the breaker did not trip.

The contacts were cleaned and the breaker returned to service after successful testing. This event was of minimal safety significance since the unit was in a refueling outage and the redundant breaker did not exhibit a similar problem during the relay surveillance tests. If during normal operation an emergency condition had occurred (loss of AC power), the emergency diesel generator would have started, but due to the failure of breaker 152-2430 to trip open, the diesel generator would have not closed onto bus 24-1. However, since this was an isolated incident, the redundant system was not affected and therefore the 2/3 diesel generator would have provided the necessary emergency power to maintain the reactor in a safe condition. This was the first occurrence of this type at Dresden.

SUPPLEMENTAL REPORT TO DIR/LER

DVR NO.	STA	UNIT	YEAR	NO.
D- 12	-	2	-	84 - 112

PART 1 TITLE OF EVENT OCCURRED

4 KV Feed Breaker Failure	12/20/84	1500
REASON FOR SUPPLEMENTAL REPORT	DATE	TIME

This supplemental report is being submitted to provide additional
information concerning possible consequences of the event.

PART 2

ACCEPTANCE BY STATION REVIEW

DATE

SUPPLEMENTAL REPORT APPROVED
AND AUTHORIZED FOR DISTRIBUTION

<u>J. Brannon</u>	<u>John M. Akers</u>
<u>3/2/85</u>	<u>3/12/85</u>
<u>Douglas Mott</u>	<u>3/12/85</u>
STATION SUPERINTENDENT	DATE



Commonwealth Edison

Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

February 25, 1985

DJS Ltr #85-211

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Supplement to Licensee Event Report #84-025-1, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(ii) and (v). This supplement is being submitted to supply additional information concerning possible consequences of this event.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

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