

LICENSEE EVENT REPORT (LER)

APPROVED OMS NO. 3180 DION
EXPIRES 8/31/85

FACILITY NAME (1) Limerick Generating Station - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 5 2				PAGE (3) 1 OF 03		
TITLE (4) Apparent Inoperability of the MSIV-Leakage Control System																
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME				DOCKET NUMBER (5)			
01	31	085	85	022	01	00	31	285					0 5 0 0 0 1 1			
OPERATING MODE (9) 2			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)													
POWER LEVEL (10) 0 0 3			20.402(a)			20.406(a)			60.73(a)(2)(H)			73.71(a)				
			20.406(a)(1)(D)			60.36(a)(1)			60.73(a)(2)(V)			73.71(a)				
			20.406(a)(1)(E)			60.36(a)(2)			60.73(a)(2)(VH)			OTHER (Specify in Abstract below and in Text, NRC Form 364-J)				
			20.406(a)(1)(M)			60.73(a)(2)(U)			60.73(a)(2)(VH)(A)							
			20.406(a)(1)(W)			60.73(a)(2)(H)			60.73(a)(2)(VH)(B)							
			20.406(a)(1)(X)			60.73(a)(2)(H)			60.73(a)(2)(A)							
LICENSEE CONTACT FOR THIS LER (12)																
NAME John C. Nagle, Engineer - Special Projects										TELEPHONE NUMBER AREA CODE 2 1 5 8 4 1 - 5 1 8 4						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 85-022

On January 30, 1985, at approximately 5:15 p.m., with Unit 1 in the startup condition and at 3.5 percent power, the feed breakers for the blowers and heaters in the Main Steam Isolation Valve-Leakage Control System (MSIV-LCS) were observed to be open during an NRC inspection. Technical Specification 3.6.1.4 requires the MSIV-LCS be operable during startup condition. With the feed breakers open, the operability of the MSIV-LCS was in question. The breakers had been left open in accordance with Surveillance Test ST-6-040-320-1. The cause of this event was the result of a discrepancy between system operating procedures and ST-6-040-320-1. Upon identification, the breakers were closed. ST-6-040-320-1 was revised to require the breakers be placed in the closed position at the completion of the surveillance test.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		-0 2 2	-0 1 0	0 2	OF	0 3	

TEXT (if more space is required, use additional NRC Form 366a (17))

Description of the Event:

On January 30, 1985, at approximately 5:15 p.m., with Unit No. 1 in the startup condition and at 3.5 percent power, during an NRC inspection, it was identified that the breakers for the blowers and heaters in the Main Steam Isolation Valve-Leakage Control System (MSIV-LCS) subsystems were open. Technical Specification 3.6.1.4 requires two independent MSIV-LCS subsystems be operable when the plant is in the startup condition. With the feed breakers open, the operability of the MSIV-LCS was in question. The breakers had been left in the open position in accordance with the Surveillance Test, ST-6-040-320-1, following performance of the Surveillance Test on December 27, 1984. The feeds to the blowers and heaters in the MSIV-LCS were closed upon identification by the NRC inspectors.

Consequences of the Event:

The MSIV-LCS is a manually initiated system following a Loss of Coolant Accident (LOCA). Special Event procedure SE-10 for a LOCA directs the operator to actuate the MSIV-LCS, but does not specifically direct how. In order to do so, the operator must use the MSIV-LCS startup procedures, S40.1.A and S40.1.B, which specifically direct the operator to close all system feed breakers including those which had been left open as the result of performing ST-6-040-320-1.

Cause of the Event:

The cause of the event was the result of a discrepancy between the system operating procedures and Surveillance Test ST-6-040-320-1.

The feed breakers for the blowers and heaters had been closed as a result of performing the checkoff lists for system procedures S40.3.A and S40.3.B. Subsequently, on December 27, 1984, ST-6-040-320-1 was performed which required that the feed breakers be left in the open position. The breakers were not repositioned since there was no cause to re-perform system procedures S40.3.A and S40.3.B.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Actions:

Upon identification of the open blower and heater feed breakers, they were immediately closed. Special Event procedure SE-10 was revised to specifically reference system procedures S40.1.A and S40.1.B in order to startup the MSIV-LCS. ST-6-040-320-1 was revised to require the feed breakers to be repositioned to the closed position at the completion of the Surveillance Test.

Previous Similar Occurrences:

None.

PHILADELPHIA ELECTRIC COMPANY

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March 12, 1985
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Docket No. 50-352

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Washington, DC 20555

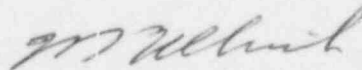
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER concerns the apparent inoperability of the Main Steam Isolation Valve-Leakage Control System.

Reference:	Docket No. 50-352
Report Number:	85-022
Revision Number:	00
Event Date:	January 30, 1985
Report Date:	March 12, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(vi). The delayed submittal of this report was the result of an ongoing investigation concerning this event which we expected to resolve prior to the due date of this LER. We regret any inconveniences the delay may have caused.

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

TE22
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Docket & Service Section (3 Copies)
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January 16, 1985