

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Joseph M. Farley - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 6 4				PAGE (3) 1 OF 2									
TITLE (4) Reactor Trip																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)										
1	0	2	6	8	4	8	4	0	1	2	0	0	1	1	2	1	8	4	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																					
1		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)									
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(c)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)									
1		20.405(a)(1)(ii)				50.38(c)(2)				<input type="checkbox"/> 50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
		20.405(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(vii)(A)													
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)													
		20.405(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME J. D. Woodard										TELEPHONE NUMBER AREA CODE 2 0 5 8 9 9 - 5 1 5 6													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs				
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

At 0930 on 10-26-84, with the unit operating at 100% power, the reactor tripped due to "general warnings" in both trains of the Solid State Protection System. This was caused by operator error during the performance of FNP-2-STP-33.0 (Solid State Protection System Train A (B) Operability Test).

The plant operators implemented FNP-2-EFP-0 (Reactor Trip or Safety Injection) and FNP-2-ESP-0.1 (Reactor Trip Response), placing the unit safely in Mode 3. All safety systems functioned as designed. Health/safety of the public was not affected.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Joseph M. Farley - Unit 2	0 5 0 0 0 3 6 4 8 4	— 0	1 2	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

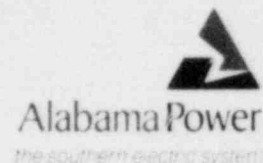
On 10-26-84, with the unit operating at 100% power, a plant operator was performing FNP-2-STP-33.0 (Solid State Protection System Train A (B) Operability Test) on the "B" train SSPS. During the performance of this procedure, a step requiring closing and locking the "A" train output and logic cabinet doors was performed in error and the "B" train doors were closed and locked instead. Continuing the procedure, the operator closed the "B" train reactor trip bypass breaker as required, resulting in a "B" train general warning. Thinking he was at the "B" train output cabinet, the operator then placed the "A" train output mode switch in test, resulting in a "A" train general warning. The reactor tripped at 0930 due to general warnings in both trains of SSPS.

The plant operators implemented FNP-2-EFP-0 (Reactor Trip or Safety Injection) and FNP-2-ESP-0.1 (Reactor Trip Response), placing the unit safely in Mode 3. All safety systems functioned as designed.

This event was caused by personnel error. The plant operator was counseled concerning the importance of adherence to procedures. FNP-2-STP-33.0 has been changed to require verification of the general warning in the train being tested before placing the output mode switch in test. To further reduce the likelihood of recurrence, both the "A" and "B" train logic and output cabinets have been labeled on the inside of the cabinet doors. This is in addition to the labels already present on the outside of the cabinets.

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R. P. McDonald  
Senior Vice President  
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November 21, 1984

Docket No. 364

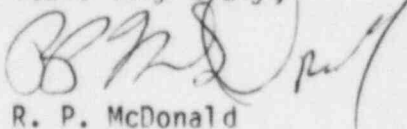
Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant, Unit 2, Licensee Event Report  
No. LER 84-012-00 is forwarded in accordance with 10CFR50.73 to provide 30  
day written notification of the occurrence.

If you have any questions, please advise.

Yours very truly,



R. P. McDonald

RPM/DSM:sam

Enclosure

xc: IE, Region II

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