NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
LaSalle County Station Unit 1	0 5 0 0 0 3 7 3	1 OF 0 3

TITLE (4)
Inadvertent Closure of Reactor Water Cleanup Outboard Isolation Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									NA	0 5 0 0 0
1 0	2 4	8 4	8 4	— 0 7 1	— 0 0	1 1	2 0	8 4		0 5 0 0 0

OPERATING MODE (9)	4	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 9 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
William Kirchhoff, extension 209	AREA CODE 8 1 5 3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
A	J M	Z 1 9 1 9	Z 9 9 1 9	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Reactor Water Cleanup (RWCU) Outboard Isolation valve inadvertently closed during the performance of a logic test surveillance on the primary containment isolation manual pushbutton for Division 1. This closure was due to an incorrect power breaker being opened to prevent actuation. The power breaker which was opened was for 1G33-Z001-4 instead of 1G33-F004. The closeness in equipment number contributed to the error. The closure did not unduly affect plant operation. The Non-Licensed Operators were instructed to closely look at equipment numbers when taking equipment out of service.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
LaSalle County Station Unit 1	0 5 0 0 0 3 7 3	8 4	— 0 7 1	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. EVENT DESCRIPTION

On 10/24/84 at 1330, the Unit 1 Reactor Water Cleanup system (RWCU, CE) Outboard Isolation Valve, 1G33-F004, inadvertently closed, due to a Group 5 Isolation Signal (JM) generated during the logic testing of the Primary Containment Isolation System (PCIS). At the time of the occurrence Unit 1 was in Cold Shutdown. LES-PC-10, Primary Containment Isolation Manual Initiation Logic Test, required that specified control power breakers to valves be opened prior to testing the PCIS Manual Initiation logic, to prevent spurious isolation of the systems during testing. These breakers were specified by the Shift Foreman on Attachment A of LES-PC-10. During the testing of the Division 1 PCIS Manual Initiation logic, the RWCU Outboard Isolation Valve, 1G33-F004, unexpectedly isolated upon PCIS initiation.

II. CAUSE

The RWCU Outboard Isolation Valve closed on the manual initiation signal because the wrong control power breaker had been opened. Attachment A of LES-PC-10 had specified that breaker A4 at MCC 134X-1 be opened for 1G33-F004, instead of the correct breaker E5 at MCC 135X-1. The reason for the wrong breaker being identified on the Attachment is due to the closeness in the equipment number of 1G33-Z001-4 which is fed from MCC 134X-1, A4, and an error made when the control power checklist was initially developed.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

Consequences of the event were minimal. Upon closure of the 1G33-F004, the RWCU system shut down as required. The equipment which had been deenergized inadvertently was the RWCU Precoat Pump, which did not affect plant operation at the time of the event. Had the plant been operating at power, no additional effect on plant operations, except loss of RWCU for a short period of time, would have been experienced.

IV. CORRECTIVE ACTIONS

NRC notification was made at 1726 on 10/24/84 of the ESF actuation.

Operating Equipment Attendents (Non-Licensed Operators) have been instructed to look at the equipment name labels identified on the MCC breakers, as well as the MCC number and breaker identified by a checklist when removing and returning equipment from service. Additional emphasis has been placed on the importance of independence of the second verification.

V. PREVIOUS OCCURRENCES

None.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
LaSalle County Station Unit 1	0 5 0 0 0 3 7 β	8 4	— 0 7 1	— 0 0	0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

VI. NAME AND TELEPHONE NUMBER OF PREPARER

William Kirchhoff, 815/357-6761, extension 209.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

November 20, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-071-00, Docket #050-373 is being submitted to your office in accordance with 10CFR 50.73.

C E Sargent
for G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

IE 22
'1