

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
D. C. COOK UNIT 2DOCKET NUMBER (2)
0 5 0 0 0 3 1 6 1 OF 0 2TITLE (4)
ASSUMPTIONS ERROR IN AUXILIARY FEEDWATER HYDRAULIC ANALYSIS

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)																	
1	0	2	5	8	4	8	4	0	2	8	0	0	1	1	2	1	8	4	D. C. COOK UNIT 1	0	5	0	0	0	3	1	5
											0	5	0	0	0												

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10)	1 0 0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)			
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)		X 50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)
NAME
A. A. BLIND
TECHNICAL ENGINEERING SUPERINTENDENT
TELEPHONE NUMBER
6 1 6 4 6 5 - 5 9 0 1
AREA CODE

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON OCTOBER 25, 1984, WITH BOTH UNITS 1 AND 2 AT 100 PERCENT RATED THERMAL POWER, A REVIEW OF THE AUXILIARY FEEDWATER SYSTEM HYDRAULIC ANALYSIS (LETTER AEP:NRC:300C, NOVEMBER 3, 1980) REVEALED AN ERROR IN THE INPUT ASSUMPTIONS.

A POWER REDUCTION WAS MADE ON BOTH UNITS TO REDUCE THE AMOUNT OF DECAY HEAT, USED IN SAFETY ANALYSIS, EQUIVALENT TO THE DISCOVERED REDUCTION IN AUXILIARY FEEDWATER FLOW. POWER WAS RETURNED TO 100 PERCENT ON BOTH UNITS AFTER THE NRC ISSUED A SAFETY EVALUATION REPORT ON BOTH UNITS ON OCTOBER 30, 1984.

A REVISED SAFETY ANALYSIS FOR UNIT 2 WILL BE INCORPORATED IN THE FSAR FOR THE NEXT ANNUAL UPDATED FSAR. THE FEEDWATER LINE BREAK WILL ALSO BE ADDRESSED IN THE UNIT 1 UPDATED FSAR EVEN THOUGH IT IS NOT PART OF THE ORIGINAL DESIGN BASIS. COMPLETION OF THESE ACTIONS WILL NOT RESULT IN AN UPDATE TO THIS LER.

8411290397 841121
PDR ADOCK 05000316
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) D. C. COOK UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 1 6 8 4 -	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4 -	0 2 8 -	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ON OCTOBER 25, 1984, WITH BOTH UNITS 1 AND 2 AT 100 PERCENT RATED THERMAL POWER, A REVIEW OF THE AUXILIARY FEEDWATER SYSTEM (IEEE SYSTEM FUNCTION IDENTIFIER = BA) HYDRAULIC ANALYSIS (LETTER AEP:NRC:300C, NOVEMBER 3, 1980) REVEALED AN ERROR IN THE INPUT ASSUMPTIONS.

A POWER REDUCTION WAS INITIATED AT 2035 HOURS ON UNIT 1 AND 2039 HOURS ON UNIT 2 AS A PRECAUTIONARY MEASURE. THE POWER REDUCTIONS WERE TAKEN TO REDUCE THE AMOUNT OF DECAY HEAT, USED IN THE SAFETY ANALYSIS, EQUIVALENT TO THE DISCOVERED REDUCTION IN AUXILIARY FEEDWATER FLOW (HEAT REMOVAL CAPABILITY).

THE ERROR REVEALED THAT THE RESULTANT FLOW RATE TO THE INTACT STEAM GENERATOR WAS 75 GPM BELOW THE VALUE USED IN THE UNIT 2 SAFETY ANALYSIS OF A FEEDWATER LINE BREAK, FSAR SECTION 14.2.8. THE ERROR IS COMMON TO BOTH UNITS; HOWEVER, UNIT 1 DOES NOT HAVE A FEEDWATER LINE BREAK ACCIDENT AS PART OF ITS DESIGN BASIS. THE MAJOR IMPACT OF THE ERROR ON THE SAFETY ANALYSIS IS DESCRIBED IN OUR LETTER AEP:NRC 0300H, OCTOBER 30, 1984.

POWER WAS RETURNED TO 100 PERCENT ON BOTH UNITS AFTER THE NRC ISSUED A SAFETY EVALUATION REPORT ON BOTH UNITS ON OCTOBER 30, 1984. THE SAFETY EVALUATION REPORT ACKNOWLEDGED OPERATOR ACTIONS TO ISOLATE THE AUXILIARY FEEDWATER FLOW TO THE FAULTED STEAM GENERATOR TEN MINUTES INTO THE EVENT ON UNIT 1, AND SHOWED SAFE OPERATION OF UNIT 2 UNDER THE ASSUMPTIONS AS IN THE CURRENT FSAR EXCEPT FOR THE REDUCTION IN AUXILIARY FEEDWATER FLOW TO THE INTACT STEAM GENERATOR.

A REVISED SAFETY ANALYSIS FOR UNIT 2 WILL BE INCORPORATED IN THE FSAR FOR THE NEXT ANNUAL UPDATED FSAR. THE FEEDWATER LINE BREAK WILL ALSO BE ADDRESSED IN THE UNIT 1 UPDATED FSAR EVEN THOUGH IT IS NOT PART OF THE ORIGINAL DESIGN BASIS.



INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
(616) 465-5901

November 21, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

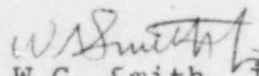
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Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10CFR50.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 84-028-0

Sincerely,


W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
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