

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Catawba Nuclear Station, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 1 3				PAGE (3) 1 OF 01										
TITLE (4) Auto Start of Motor Driven Auxiliary Feedwater Pumps																								
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)											
1	0	1	2	8	4	8	4	0	1	7	0	0	1	1	6	8	4	0	5	0	0	0	0	0
OPERATING MODE (9) 4		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																						
POWER LEVEL (10) 0 0 0		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)										
		20.405(a)(1)(i)				50.36(e)(1)				50.73(a)(2)(v)				73.71(c)										
		20.405(a)(1)(ii)				50.36(e)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)										
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)														
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)														
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)														
LICENSEE CONTACT FOR THIS LER (12)																								
NAME Roger W. Ouellette, Assistant Engineer-Licensing										TELEPHONE NUMBER 7 0 4 3 7 3 - 7 5 3 0														
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR								
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												NO		1	2	0	1	8	4					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 21, 1984, at 0635 hours, Motor Driven Auxiliary Feedwater (CA) Pumps 1A and 1B started due to a trip signal on both Main Feedwater (CF) Pump Turbines. Unit 1 was in Mode 4 (Hot Shutdown) with the CA System in standby alignment in preparation for entering into Mode 3 (Hot Standby).

Work was being conducted on Valve 1CA-151 (S/G 1C BYP to CA Nozzle). To allow for this work, the Nuclear Control Operator (NCO) was feeding Steam Generator (S/G) 1C by the main feedwater nozzle through valve 1CF-051 (S/G 1C CF CONT. ISOL). Prior to shift turnover, the NCO started feeding S/G 1C through its normal alignment by closing 1CF-051 and feeding into the CA nozzle through valve CA-151. Shortly after this, the temporary air supply was removed from 1CA-151. At this point, valve 1CA-151 failed closed, isolating S/G 1C. Twenty minutes later S/G 1C had a Low Level Deviation (5% low level) Alarm. The NCO immediately opened valve 1CF-051, feeding water quickly to the main feedwater nozzle through Throttle Valve 1CF-048 (S/G 1C CF BYP CTRL). Both Main Feedwater Pumps were reset and windmilling when flow to the Main Feed Nozzle was aligned. Feeding water rapidly into S/G 1C appears to have put the Condensate (CM) System and CF in a pressure transient, developing low suction pressure at the CF Pumps. The low suction pressure alarm initiated the CF Pump Turbine Trip Signal, causing the Motor Driven CA Pumps to auto start.

The cause of the incident is tentatively unknown. A follow-up report will be issued no later than 12/1/84. Additional investigative testing is being performed and will be evaluated prior to issuance of the follow-up report.

DUKE POWER COMPANY

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November 16, 1984

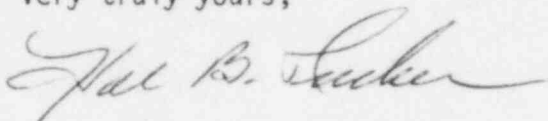
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U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Subject: Catawba Nuclear Station, Unit 1  
Docket No. 50-413

Gentlemen:

Pursuant to 10 CFR 50.73 Section (a) (1) and (d), attached is Licensee Event Report 413/84-17 concerning the auto start of motor driven auxiliary feedwater pumps. This event was considered to be of no significance with respect to the health and safety of the public.

Very truly yours,



Hal B. Tucker

RWO:slb

Attachment

cc: Mr. James P. O'Reilly, Regional Administrator  
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