



BOSTON EDISON

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

E. Thomas Boulette, PhD
Vice President Nuclear Operations
and Station Director

August 12, 1992
BECo Ltr. #92-098

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

License No. DPR-35
Docket No. 50-293

Subject: July 1992 Monthly Report

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

E. Thomas Boulette
E. Thomas Boulette

WJM/bal

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region 1
U.S. Nuclear Regulatory Commission
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Senior Resident Inspector

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
UNIT Pilgrim I
DATE August 12, 1992
COMPLETED BY W. Munro
TELEPHONE (508) 747-8474

MONTH July 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>663</u>	17	<u>659</u>
2	<u>663</u>	18	<u>641</u>
3	<u>662</u>	19	<u>520</u>
4	<u>663</u>	20	<u>662</u>
5	<u>663</u>	21	<u>662</u>
6	<u>665</u>	22	<u>660</u>
7	<u>664</u>	23	<u>660</u>
8	<u>663</u>	24	<u>660</u>
9	<u>663</u>	25	<u>660</u>
10	<u>664</u>	26	<u>623</u>
11	<u>662</u>	27	<u>581</u>
12	<u>662</u>	28	<u>662</u>
13	<u>661</u>	29	<u>660</u>
14	<u>662</u>	30	<u>660</u>
15	<u>661</u>	31	<u>660</u>
16	<u>658</u>		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE August 12, 1992
 COMPLETED BY W. Munro
 TELEPHONE (508) 747-8474

OPERATING STATUS

Notes

1. Unit Name Pilgrim 1
2. Reporting Period July 1992
3. Licensed Thermal Power (Mwt) 1598
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe) None

10. Reasons For Restrictions, If Any N/A

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744.0	5111.0	172199.0
12. Number Of Hours Reactor Was Critical	744.0	4733.6	103094.2
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	744.0	4681.5	99184.4
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated(MWH)	146664.0	9173952.0	172878432.0
17. Gross Electrical Energy Generated(MWH)	504600.0	3166130.0	52372544.0
18. Net Electrical Energy Generated (MWH)	485551.0	3047979.0	56098927.0
19. Unit Service Factor	100.0	91.6	57.6
20. Unit Availability Factor	100.0	91.6	57.6
21. Unit Capacity Factor (Using MDC Net)	97.4	89.0	48.6
22. Unit Capacity Factor (Using DER Net)	99.6	91.0	49.7
23. Unit Forced Outage Rate	0.0	8.4	12.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			
Midcycle outage - October 1992 Approximately 35 days			

25. If Shut Down At End Of Report Period, Estimated Date of Startup N/A

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: April 3, 1993
3. Scheduled date for restart following next refueling: June 8, 1993
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1991 refueling outage was of the same design as loaded in the previous outage and consisted of 168 assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1489 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
 - (b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 831 fuel assemblies.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Operational Summary for July 1992

The unit started the reporting period at approximately 100 percent core thermal power (CTP). On July 18, at approximately 2300 hours, reactor power was reduced to facilitate a thermal backwash of the main condenser, and perform selective maintenance. Following the backwash, reactor power was raised to 100 percent CTP where it was maintained until July 26 when at 2123 hours power was reduced to 40 percent CTP when the "A" Recirculation Pump Motor Generator Set drive motor breaker tripped causing a trip of the "A" Recirculation Pump. On July 27 at approximately 1300 hours reactor power was returned to 100 percent CTP where it was essentially maintained for the remainder of the reporting period. Weekly control rod exercises were performed on July 4, 11, 19, and 25.

Safety Relief Valve Challenges
Month of July 1992

Requirement: NUREG-0737 I.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

MONTH July, 1992

PILSRIM NUCLEAR POWER STATION
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Salt Service Water (SSW) System	Motor Operated Valve MO-3813	During performance of Surveillance Procedure 8.5.3.1 "SSW MOV Operability" MO-3813 failed to indicate full closure. (PR 92.9118)	Lack of lubrication on the spline and spline adaptor.	Lubricated spline and spline adaptor.	Inspect valves (MO-3800, MO-3805 MO-3806, MO-3808) and lubricate as required.	N/A
Salt Service Water (SSW) System	"D" SSW Pump P-208D	High vibration; pump in "Required Action Range" (PR92.9118)	Less than optimum design.	Added weights on the motor coupling to compensate for motor imbalance.	NED preparing new improved design for SSW pumps. CJSa 92-013 in review process.	N/A
Diesel Generators and Auxiliaries	A" Emergency Diesel Generator starting air Check Valve 47-CK-101C.	Check valve failed in open position.	Corrosion buildup in check valve.	Disassembled Check Valve, cleaned internals replaced valve seat. Reassembled valve and re-installed. Performed Post Work Test 8.9.1.2 successfully.	Established PM program for all Emergency Diesel Generator air system check valves. PDC 91-08 Diesel Generator Air Start Piping to be implemented during MCO-92.	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293

NAME PILGRIM 1

DATE August 12, 1992

COMPLETED BY W. MUNRO

TELEPHONE 508 747-8474

REPORT MONTH JULY 1992

NO.	DATE	TYPE1	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT#	SYSTEM CODE4	COMPONENT CODE5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
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There were no unit shutdowns or significant power reductions during the reporting period required to be reported.

1	2	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)