

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Susquehanna Steam Electric Station - Unit 2.

DOCKET NUMBER (2)

0 5 0 0 0 3 8 8

PAGE (3)

1 OF 0 1

TITLE (4)

Unanticipated ESF Actuation - RHR Shutdown Cooling.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)				
01	29	85	85	006	00	02	28	85		0 5 0 0 0 0				
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)						73.71(b)					
3			20.402(b)						X 50.73(a)(2)(iv)			73.71(c)		
POWER LEVEL (10)			20.406(a)(1)(i)						50.73(a)(2)(v)			OTHER (Specify in Abstract below and in Text, NRC Form 368A)		
0 0 0			20.406(a)(1)(ii)						50.73(a)(2)(vi)					
			20.406(a)(1)(iii)						50.73(a)(2)(vii)(A)					
			20.406(a)(1)(iv)						50.73(a)(2)(vii)(B)					
			20.406(a)(1)(v)						50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME

L.A. Kuczynski - Nuclear Plant Specialist, III

TELEPHONE NUMBER

AREA CODE

7 1 1 7 5 4 1 2 1 3 1 7 5 1 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	BIO	IPSB	081	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)

X NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces - i.e., approximately fifteen single space typewritten lines) (16)

On January 29, 1985, with the Unit in Hot Shutdown, the Residual Heat Removal (RHR) System shutdown cooling suction inboard isolation valve closed unexpectedly. This valve is part of the Primary Containment Isolation System, which is an Engineered Safety Feature. The calibration of the pressure switch, which provides a reactor vessel high pressure permissive signal to the valve, was checked. The switch's 'As Found' setpoint, while low, was within the 'As Found' tolerance band. The switch was recalibrated nevertheless, and shutdown cooling was established. The switch may be replaced pending an evaluation of the system operating restrictions imposed by the use of this type of switch in this application. Additionally, this event will be reviewed by all licensed operators.

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PDR ADOCK 05000386  
S PDR



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

March 4, 1985

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
CORRECTION - LICENSEE EVENT REPORT  
85-006-00, DATED FEBRUARY 28, 1985  
ER 100450 FILE 841-23  
PLAS-049

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Docket No. 50-388  
License No. NPF-22

Please note a typographical error on Page 1 of 01 of the Licensee Event Report (LER) NRC Form 366, Item (6), LER Number. The sequential number should read 006 instead of 007. A corrected copy of Licensee Event Report 85-006-00 is attached.

A.P. Piemontese

/pjg

IE22  
1/1



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February 28, 1985


U.S. Nuclear Regulatory Commission  
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Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 85-006-00  
ER 100450 FILE 841-23  
PLAS-046

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Docket No. 50-388  
License No. NPF-22

Attached is Licensee Event Report 85-006-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that the Unit experienced an unanticipated actuation of an Engineered Safety Feature when a containment isolation valve closed.

  
H.W. Keiser  
Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley  
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