



Commonwealth Edison

Quad Cities Nuclear Power Station
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GCT-92-33

August 4, 1992

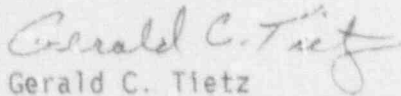
U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
Envelope Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of July 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION


Gerald C. Tietz
Technical Superintendent

GCT/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JULY 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One was in coast down for the month of July in preparation for the upcoming refuel outage.

Only two significant load reductions were ordered by Chicago Load Dispatch (CLD) in July; July 3 to 450 MWe, and July 21 to 500 MWe.

Numerous other load reductions of less than 20% were performed per CLD but not reported.

B. Unit Two

Quad Cities Unit Two reduced power to 200 MWe on July 10 in preparation for a Main Steam Isolation Valve (MS V) room entry. A one half group one isolation had been received from a failed main steam tunnel temperature switch.

Chicago Load Dispatch requested the following load reductions for Unit Two for the month of July;

<u>Date</u>	<u>Load</u>
7-2-92	490 MWe
7-3-92	450 MWe
7-4-92	470 MWe
7-5-92	450 MWe
7-6-92	450 MWe
7-7-92	470 MWe
7-9-92	500 MWe
7-9-92	497 MWe
7-12-92	495 MWe
7-13-92	600 MWe
7-14-92	450 MWe
7-16-92	551 MWe
7-16-92	478 MWe
7-20-92	580 MWe
7-21-92	485 MWe
7-22-92	535 MWe
7-23-92	462 MWe
7-24-92	600 MWe
7-26-92	450 MWe
7-27-92	550 MWe
7-28-92	550 MWe
7-29-92	550 MWe
7-30-92	660 MWe
7-31-92	650 MWe

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Amendment No. 135 was issued on July 24, 1992 to Facility Operating License DPR-29 and Amendment No. 131 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station. These amendments consist of administrative changes to revise the types of procedures that require review by the onsite review and investigation function (OnSR&IF), specifies the level of review and approval for procedures governed by the proposed technical review and control process, and clarifies the authority assigned to the OnSR&IF.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q01996	1140	Replace burned out light bulb.	As Left: Replaced bulb.
Q02060	6641	Recalibrate or replace temperature switch.	As Left: Installed new switch.
Q02102	2300	Adjust HPCI MGU high speed stop limit switch.	As Left: Adjusted approximately 3/32" from the mechanical stop.

UNIT 2 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q02103	2202	Investigate and repair panel accumulator monitor.	As Found: Found short in level switch wiring. As Left: Replaced level switch.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.2 of the Technical Specifications.

There were no Licensee Event Reports for Unit 1 and Unit 2 for this reporting period.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE August 5, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 070192
1. REPORTING PERIOD: 2400 073192 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	4715.60	141226.70
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3421.9
7. HOURS GENERATOR ON LINE	744.00	4675.20	136906.30
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	909.2
9. GROSS THERMAL ENERGY GENERATED (MMH).....	1481409.60	10918531.2	294985554.20
10. GROSS ELECTRICAL ENERGY GENERATED (MMH).....	467068.0	3529401.0	95603737.0
11. NET ELECTRICAL ENERGY GENERATED (MMH).....	444092.0	3398988.0	90108356.0
12. REACTOR SERVICE FACTOR.....	100.00	92.26	79.35
13. REACTOR AVAILABILITY FACTOR.....	100.00	92.26	81.27
14. UNIT SERVICE FACTOR	100.00	91.47	76.92
15. UNIT AVAILABILITY FACTOR	100.00	91.47	77.43
16. UNIT CAPACITY FACTOR (Using MDC)	77.62	86.43	65.84
17. UNIT CAPACITY FACTOR (Using Design MWe)	75.65	84.22	64.17
18. UNIT FORCED OUTAGE RATE	0.00	8.54	5.82

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE August 5, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 070192
1. REPORTING PERIOD: 2400 073192 GROSS HOURS IN REPORTING PERIOD: 714
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY, 769
DESIGN ELECTRICAL RATING (MWe-Net): 782
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):
5. NUMBER OF HOURS REACTOR WAS CRITICAL THIS MONTH YR TO DATE CUMULATIVE
6. REACTOR RESERVE SHUTDOWN HOURS 0.0 0.0 2985.8
7. HOURS GENERATOR ON LINE 744.00 1948.55 131968.45
8. UNIT RESERVE SHUTDOWN HOURS 0.0 0.0 702.9
9. GROSS THERMAL ENERGY GENERATED (MWH)..... 1651154.40 4032271.20 284116157.20
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)..... 535651.00 1301478.00 91231668.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)..... 510633.00 1216824.00 86350999.00
12. REACTOR SERVICE FACTOR..... 100.00 39.51 76.81
13. REACTOR AVAILABILITY FACTOR..... 100.00 39.51 78.50
14. UNIT SERVICE FACTOR 100.00 38.12 74.80
15. UNIT AVAILABILITY FACTOR 100.00 38.12 75.20
16. UNIT CAPACITY FACTOR (Using MDC) 69.26 30.96 63.65
17. UNIT CAPACITY FACTOR (Using Design MWe) 87.00 30.17 62.04
18. UNIT FORCED OUTAGE RATE 0.0 0.0 7.98
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE August 5, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH July 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	672
2.	660
3.	604
4.	427
5.	442
6.	598
7.	658
8.	654
9.	654
10.	645
11.	643
12.	623
13.	634
14.	616
15.	590
16.	618

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	609
18.	602
19.	599
20.	610
21.	598
22.	524
23.	553
24.	577
25.	561
26.	570
27.	588
28.	576
29.	605
30.	600
31.	578

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE August 5, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH July 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	779
2.	771
3.	544
4.	529
5.	620
6.	658
7.	660
8.	678
9.	724
10.	454
11.	557
12.	713
13.	718
14.	749
15.	698
16.	712

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	681
18.	772
19.	777
20.	778
21.	715
22.	668
23.	679
24.	679
25.	710
26.	646
27.	696
28.	706
29.	715
30.	751
31.	727

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME Unit One

DATE August 6, 1992

REPORT MONTH July, 1992

COMPLETED BY Mat. Hew Benson

TELEPHONE 399-654-2241

NO.	DATE	TYPE S OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-29	7-3-92	S	58.7	F	5	- - -	-	-	Load Drop per Chicago Load Dispatcher
92-30	7-21-92	S	9.3	F	5	- - -	-	-	Load Drop per Chicago Load Dispatcher

APPENDIX F UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Unit Two

DATE August 6, 1992

COMPLETED BY Matthew Benson

TELEPHONE 309-654-2241

REPORT MONTH July, 1992

NO.	DATE	TYPE P OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-17	7-02-92	S	9.5	F	S	-	-	-	Load Drop Per Chicago Load Dispatcher
92-18	7-03-92	S	11.3	F	S	-	-	-	"
92-19	7-04-92	S	9.3	F	S	-	-	-	"
92-20	7-05-92	S	8.1	F	S	-	-	-	"
92-21	7-06-92	S	7.8	F	S	-	-	-	"
92-22	7-07-92	S	7.3	F	S	-	-	-	"
92-23	7-09-92	S	3.7	F	S	-	-	-	"
92-24	7-09-92	S	8.9	S	S	-	-	-	"
92-25	7-10-92	F	19.8	A	S	-	-	-	Failed MSL Temperature Switch
92-26	7-12-92	S	4.2	F	S	-	-	-	Load Drop Per Chicago Load Dispatcher
92-27	7-13-92	S	6.0	F	S	-	-	-	"
92-28	7-14-92	S	7.2	F	S	-	-	-	"
92-29	7-16-92	S	6.5	F	S	-	-	-	"
92-30	7-16-92	S	7.0	F	S	-	-	-	"
92-31	7-20-92	S	6.7	F	S	-	-	-	"
92-32	7-21-92	S	7.0	F	S	-	-	-	"
92-33	7-22-92	S	7.3	F	S	-	-	-	"
92-34	7-23-92	S	6.9	F	S	-	-	-	"
92-35	7-24-92	S	6.8	F	S	-	-	-	"
92-36	7-26-92	S	8.3	F	S	-	-	-	"
92-37	7-27-92	S	7.8	F	S	-	-	-	"
92-38	7-28-92	S	6.4	F	S	-	-	-	"
92-39	7-29-92	S	5.7	F	S	-	-	-	"
92-40	7-30-92	S	3.8	F	S	-	-	-	"
92-41	7-31-92	S	4.0	S	S	-	-	-	"

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
PERFORMED ON UNIT 1 & 2 CONTROL
ROD DRIVES, FROM 1-1-92 TO 12-31-92

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5 0.375	20 0.900	50 2.00	90 3.5		
						7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
2-19-92	2	0.28	0.67	1.43	2.48	J-2 2.55	Drive Replacement (J-2), Scram Valve N-7
2-20-92	1	0.32	0.69	1.45	2.45	N-5 2.45	Scram Valve Work N-5
5-12-92	177	0.31	0.69	1.47	2.58	L-13 3.43	Start-up Scram Timing Unit Two
6-02-92	1	0.31	0.65	1.37	2.35	N-5 2.35	Scram Valve Work
6-26-92	2	0.27	0.62	1.32	2.32	C-13 2.26	ACCUM C-13, N-7
7-13-92	88	0.28	0.63	1.35	2.36	K-13 2.79	Unit 1 "B" Sequence
7-22-92	1	0.29	0.64	1.36	2.41	D-2 2.43	127 Diaphragm D-2 Unit 2

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q1 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 9-20-92
3. Scheduled date for restart following refueling: 12-12-92
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
 1. Modification to HPCI turbine exhaust steam line.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
 1. 06/30/92
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1405
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-532
Revision 2
October 1989

1. Unit: Q2 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 03/06/93
3. Scheduled date for restart following refueling: 06/05/93
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
NOT AS YET DETERMINED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 2439
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLR1	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SFM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center