AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE November 9, 1984
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

MONTH October, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	483.5	17	484.4
2	484.9	18	485.0
3	485.7	19	484.8
4	485.4	20	485.1
5	485.1	21	485.1
6	485.2	22	484.9
7	485.0	23	484.8
8	485.0	24	485.0
9	483.7	25	483.5
10	483.3	26	484.1
11	483.1	27	485.3
12	483.6	28	484.9
13	484.0	29	484.8
14	483.8	30	484.8
15	483.8	31	484.9
16	484.5		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-285
DATE November 9, 1984
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: October, 1984
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   N/A
9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

Notes

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	745.0	7,320.0	97,322.0
12. Number Of Hours Reactor Was Critical	745.0	4,243.1	74,137.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.0
14. Hours Generator On-Line	745.0	4,136.5	73,539.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,110,458.1	5,853,527.9	92,613,241.6
17. Gross Electrical Energy Generated (MWH)	377,874.0	1,920,266.0	30,237,835.0
18. Net Electrical Energy Generated (MWH)	360,987.8	1,825,814.4	28,905,683.1
19. Unit Service Factor	100.0	56.5	75.6
20. Unit Availability Factor	100.0	56.5	75.6
21. Unit Capacity Factor (Using MDC Net)	101.4	52.2	64.7
22. Unit Capacity Factor (Using DER Net)	101.4	52.2	62.4
23. Unit Forced Outage Rate	0.0	0.4	3.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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DOCKET NO. 50-285
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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1984

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns or power reductions during the month of October, 1984.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending October, 1984 .

1. Scheduled date for next refueling shutdown. — October 1985

2. Scheduled date for restart following refueling. — December 1985

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      Technical Specification change to accommodate increased radial peaks due to further reduction in radial leakage.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. —

   c. If no such review has taken place, when is it scheduled? —

4. Scheduled date(s) for submitting proposed licensing action and support information. — September 1985

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — Methodology Changes June 1985

6. The number of fuel assemblies:
   a) in the core — 133 assemblies
   b) in the spent fuel pool — 305 "
   c) spent fuel pool storage capacity — 729 "
   d) planned spent fuel pool storage capacity — May be increased via fuel pin consolidation "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1996

Prepared by J K Gasper Date November 1, 1984

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

October, 1984
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power through the month of October, 1984. The Fort Calhoun staff participated in the annual emergency exercise on October 24, 1984. A Shift Supervisor retired effective October 1, 1984.

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
84-019	VIAS Actuation (RM-061).
84-008 R1	Steam Generator Tube Rupture, Supplement.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-VA-80	Hydrogen Purge System Test. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure only checks operability of fans and cleanliness of the filters.
SP-FAUD-1	Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (continued)

Procedure	Description
SP-VLPM-1	Vibration and Loose Parts Monitoring Core Barrel Motion. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it merely allows for recording of vibration signatures corresponding to the nuclear instrumentation channels in an effort to gather and interpolate data which could possibly identify excessive core barrel motion. At no time during the performance of this test was any equipment important to safety affected.
SP-VLPM-3	Vibration and Loose Parts Monitoring Core Barrel Motion. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it merely allows for recording of vibration signatures corresponding to the nuclear instrumentation channels in an effort to gather and interpolate data which could possibly identify excessive core barrel motion. At no time during the performance of this test was any equipment important to safety affected.

E. RESULTS OF LEAK RATE TESTS

None

F. CHANGES IN PLANT OPERATING STAFF

Mr. Robert F. Johnston retired as Shift Supervisor effective October 1, 1984.

G. TRAINING

Non-licensed operators continued individual training as part of the initial operator training program during the month of October, 1984. Licensed operators attended simulator requalification training at Combustion Engineering's simulator in Windsor, Connecticut. Emergency plan training was completed for plant staff and crafts in preparation for the annual emergency exercise held in October. Other training was conducted per annual schedule. Special training was conducted for NRC license candidates in preparation for an exam to be administered in November, 1984.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Package	Description/Analysis
Amendment No. 85	This amendment revises the technical specifications to include more formal administrative requirements on limiting overtime and reporting of pressurizer safety valve and relief valve challenges and failures.

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

November 12, 1984
LIC-84-387

Mr. Richard C. DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

October Monthly Operating Report

Please find enclosed ten (10) copies of the October Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

R. L. Andrews
Division Manager
Nuclear Production

RLA/TPM/dao:2614

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
NRC File

IE24
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