AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date Nov. 10,1984
Telephone 609-935-6000
Extension 4455

Completed by J. P. Konafalvy

Month October 1984

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	23
7	0	23	0
8	0	24	304
9	0	25	337
10	0	26	465
11	0	27	626
12	0	28	678
13	0	29	836
14	0	30	311
15	0	31	352
16	0		

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IE24
1/1

OPERATING DATA REPORT

Docket No. 50-272
Date Nov. 10, 1984
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	October 1984	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1135	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	745	7320	64345
12. No. of Hrs. Reactor was Critical	368.8	1606.4	34757.6
13. Reactor Reserve Shutdown Hrs.	0	54.5	3088.4
14. Hours Generator On-Line	205.6	1403.4	33181.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	389047	4189070	100008441
17. Gross Elec. Energy Generated (MWH)	104040	1385420	33000520
18. Net Elec. Energy Generated (MWH)	77715	1268001	31239313
19. Unit Service Factor	27.6	19.2	51.6
20. Unit Availability Factor	27.6	19.2	51.6
21. Unit Capacity Factor (using MDC Net)	9.7	16.1	45.0
22. Unit Capacity Factor (using DER Net)	9.6	15.9	44.5
23. Unit Forced Outage Rate	72.4	71.8	33.3

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

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UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH October 1984

Docket No. 50-272
Unit Name Salem No.1
Date Nov. 10,1984
Telephone 609-935-6000
Extension 4455

Completed by J.P. Ronafalvy

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
84-180	9-10	F	207	A	4	-	RB	CRDRVE	Nuclear Other Control Rod Drive Problem
84-182	10-9	S	200.6	B	4	-	RC	ZZZZZZ	Nuclear Core Physics Test
84-184	10-17	F	100.2	A	4	-	HA	XXXXXX	Seal Oil System and Seals Generator
84-186	10-22	F	1.2	B	4	-	HA	ZZZZZZ	Turbine Overspeed Trip Test
84-188	10-22	F	30.4	A	3	-	HA	INSTRU	Turbine Instruments
84-190	10-24	S	32.8	B	5	-	RC	ZZZZZZ	Nuclear Core Physics Test
84-192	10-25	F	22.7	A	5	-	HE	PUMPXX	Steam Generator Feed Pump Problems
84-194	10-28	F	5.9	A	5	-	HH	PUMPXX	Condensate/ Hotwell Pumps
84-196	10-28	F	51.1	A	5	-	HA	XXXXXX	Loss of Vacuum/High Back Pressure
84-200	10-30	F	7.2	A	5	-	CB	INSTRU	Reactor Coolant Pump Instrumentation
84-202	10-30	F	27.0	A	5	-	HA	XXXXXX	Loss of Vacuum/High Back Pressure

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH October 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: November 10, 1984
COMPLETED BY: J. Ronatalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1ET-1166	Incore Instrumentation	Disconnect the P-250 computer from the incore flux mapping system.
1EC-1231	Waste Disposal Liquid	Add break flanges to the casing drain lines of #11 and #12 Reactor Coolant Drain Tank pump casing.
1EC-1641	Containment Ventilaton	Install a 10" 'T' connection between 1VC5 and the 10" Auxiliary Building ducting.
1EC-1665	Component Cooling	Retube No. 11 Component Cooling Heat Exchanger with titanium tubes (or suitable available material).
1EC-1728	Chemical and Volume Control	Replace the piping downstream of the 1CV45 and 1CV50 valves, located on #11 and #12 C/SI pump's casing drain lines, with piping IAW Pipe Spec 496. Install 1500# blind flanges at the end of these lines. Fabricate spool pieces to connect the casing drain lines to their respective floor drain lines. The spool pieces are to be used during maintenance on the pumps and are to be in accordance with Spec.
1EC-1735	Security System	Install high mast (100') lighting fixtures and upgrade existing yard and perimeter lighting.

MAJOR PLANT MODIFICATIONS
REPORT MONTH October 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: November 10, 1984
COMPLETED BY: J. Ronatalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1745	Main Generator-Hydrogen	Add additional piping in the main generator and between the generator and hydrogen dryer to provide adequate differential pressure across the hydrogen dryer.
1EX-1759	SEC (R130)	Provide temporary instrumentation to various inputs to the 1C SEC.
1EC-1809	Reactor Coolant-Pump Shaft Vibration Monitors	Modify the shaft vibration monitor system to eliminate erroneous vibration monitor readings.
1EC-1817	Diesel Generator E300	Replace existing transformer T54 in the exciter regulator cubicle with a new type.
1EC-1849	Main Steam	Install additional steam stop valve hydraulic control bypass valve trouble light on control console for each of four valves 11-14MS167.
1ET-1856	Main Generator	Loop test.
1EC-1862	Safety Injection	Replace existing lube oil coolers on 11 and 12 SI Pumps with coolers of similar design characteristics but upgraded materials of constructoin. Upgraded coolers will have outer tubesheets and tubes consisting of titanium material.

MAJOR PLANT MODIFICATIONS
REPORT MONTH October 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: November 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1870	Radiation Monitoring	Provide an area radiation monitor in the Electrical Penetration Area with indication in the Control Room with a range of 10^{-1} to 10^{4} R/HR.
1EC-1907	Safety Injection	Replace existing SI throttling valves (i.e., 11-14SJ16) with needle valves which are designed for metering flow. An additional globe valve will be installed in series with each SJ16, and will be used as an isolation valve during shutdown.
1EC-1956	Pressurizer System	Modify Control Circuit to 1PR6 and 1PR7 valves.
1SC-1167	Reactor Coolant	Change reactor coolant flow transmitter from Fisher Porter model #10B2496PB to new model and type specified by Engineering.
1SC-1331	Chilled Water	Install an additional suction gauge no greater than 40 PSI for 11 and 12 Chilled Water Pumps.
1SC-1378	#1 Main Generator	Install imbedded slot temperature detectors in #1 Main Generator.

MAJOR PLANT MODIFICATIONS
REPORT MONTH OCTOBER 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: November 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1ET-1166	This change permits a portable computer to obtain data from non-safety grade instrumentation. The instrumentation is used to obtain periodic test data (flux map of the core) and is not involved in process control. No unreviewed safety or environmental questions are involved.
1EC-1231	This change does not alter the original design concept of the piping system in any way for the Reactor Coolant Drain Tank Pump. Also, this change does not alter the Technical Specification or the FSAR and will not increase the liquid effluent discharge from the Station. No unreviewed safety or environmental questions are involved.
1EC-1641	This design modification will not affect the ability of valve 1VC5 to perform its Containment isolation requirement. Also, the design modification does not create the possibility for an accident or malfunction of a different type than any previously evaluated and does not reduce the margin of safety defined in the basis of any Technical Specification. No unreviewed safety or environmental questions are involved.
1EC-1665	This design change involves a change of materials to an upgraded type. This modification will not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved.
1EC-1728	This change only involves addition of a revised piping diagram. No unreviewed safety or environmental questions are involved.
1EC-1735	This change only involves an upgrade of the yard lighting system. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1745	This design change involves the addition of small bore piping in the Main Generator Hydrogen System to facilitate adequate hydrogen flow. It does not affect Plant operating procedures. No change to the FSAR or the Technical Specifications is required. No unreviewed safety or environmental questions are involved.
1EX-1759	The implementation of this DCR requires the addition of a small amount of fire retardent wood. The area is fire protected. The amount of wood is below the design basis for fire protection. No unreviewed safety or environmental questions are involved.
1EC-1809	The implementation of this DCR involves penetration of a fire barrier. Instructions are included for proper resealing to maintain the required hourly fire rating. No unreviewed safety or environmental questions are involved.
1EC-1817	The replacement component meets or exceeds the design of the old part. Qualification includes requirements of IEEE 344 for seismic events. No unreviewed safety or environmental questions are involved.
1EC-1849	The intended function of the system remains unchanged. No unreviewed safety or environmental questions are involved.
1ET-1856	This DCR is part of the Westinghouse inspection, overhaul and rewind of the Unit 1 Main Generator. No unreviewed safety or environmental questions are involved.
1EC-1862	This DCR documents the replacement of the SI Pump Lube Oil Coolers with upgraded materials. The new coolers are seismically qualified to seismic I criteria and the ASME Section III Class 3 - 1981 Summer Addends for the tube side only. The shell side is classified to the 1980 ASME Section VIII Code Division I. This DCR does not increase the potential for an accident nor does it degrade the integrity of the Service Water System and the Lube Oil System for the bearings on the SI Pumps. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1870	This DCR adds a radiation monitor to determine the magnitude of a release of radioactive material in the electrical penetration area. Information from this monitor is not required for the safe shutdown of the Unit. No unreviewed safety or environmental questions are involved.
1EC-1907	This design change replaces the boron injection path throttle valves with ones of a more applicable design. Also, additional valves for isolation are included in this DCR. This design does not alter the intent of the Safety Injection System. It does not increase the consequences of an event, nor the likelihood of an occurrence. No unreviewed safety or environmental questions are involved.
1EC-1956	This design change involves rewiring for enhanced valve operation as recommended by the manufacturer. No unreviewed safety or environmental questions are involved.
1SC-1167	This DCR involves replacement of an existing transmitter. No unreviewed safety or environmental questions are involved.
1SC-1331	This DCR has installed a new gauge as per ASME Section XI. No unreviewed safety or environmental questions are involved.
1SC-1378	This DCR describes the addition of forty-eight (48) thermocouples for trending information of the generator. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
949975	MD	1	#12 CHARGING PUMP
			FAILURE DESCRIPTION: INSPECT CASING ON PUMP; EVALUATE CLADDING CRACK.
			CORRECTIVE ACTION: REPLACED PUMP WITH NEW PUMP.
84-09-27-103-6	SMD	1	#15 CFCU
			FAILURE DESCRIPTION: SERVICE WATER LEAK ON MOTOR COOLER LINE.
			CORRECTIVE ACTION: WELDED NEW FLANGE, PIPE, AND ELBOW; INSTALLED NEW PIPING AND GASKETS
84-09-21-038-0	SMD	1	#11 CFCU SERVICE WATER
			FAILURE DESCRIPTION: 3/4" HEADER HAS LEAK ON THE COIL SIDE OF 11SW248 (VENT VALVE)
			CORRECTIVE ACTION: GROUND OUT FLANGE AND PIPE; REWELDED NEW PIPE AND FLANGE; INSTALLED NEW GASKET
0099129183	SMD	1	#11 CFCU
			FAILURE DESCRIPTION: SW LEAK FROM TELLTALE ON MOTOR COOLER (WATER SPRAYING WITH UNIT IN SERVICE)
			CORRECTIVE ACTION: INSTALLED NEW COOLING COIL

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
84-09-27-114-1	SMD	1	#12 CFCU	
			FAILURE DESCRIPTION:	PIPE UPSTREAM OF THE 12SW405 AND THE FLOW TAPS ON THE OUTLET OF THE CFCU IS LEAKING.
			CORRECTIVE ACTION:	GROUND OUT HOLE IN LINE TO SOUND METAL; PAD WELDED AS PER ENGINEERING
84-09-20-028-1	SIC	1	BIT RECIRCULATION FLOW INST.	
			FAILURE DESCRIPTION:	NO OHA ON LOW FLOW; FACEPLATE IS BROKEN (BIT INOPERABLE)
			CORRECTIVE ACTION:	REPLACED FLOAT BODY ASSEMBLY, FLOAT GUIDE, RETAINING RING FLOAT, RETAINING RING TUBE, AND FLEXITALLIC GASKET
0099104776	SMD	1	BIT RECIRCULATION LINE	
			FAILURE DESCRIPTION:	THE LINE IS CLOGGED BETWEEN VALVES 1SJ79 AND CV161
			CORRECTIVE ACTION:	CUT LINES, CLEARED BLOCKAGE, AND REWELDED LINE
943625	NCS	1	VALVE 1SA114	
			FAILURE DESCRIPTION:	VALVE FAILED LEAK RATE TEST
			CORRECTIVE ACTION:	LAPPED AND BLUE CHECKED VALVE AND REPLACED BONNET GASKET
943684	NCS	1	VALVE 11SS908	
			FAILURE DESCRIPTION:	VALVE LEAKED DURING TESTING
			CORRECTIVE ACTION:	RENEWED PACKING

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-06-07-990-8	NCS	1	VALVE 13SS182
			FAILURE DESCRIPTION: VALVE FAILED LEAK RATE TEST
			CORRECTIVE ACTION: REPLACED PACKING; BLUE CHECKED, LAPPED SEAT, AND LAPPED PLUG
943760	NCS	1	VALVE 1NT26
			FAILURE DESCRIPTION: VALVE FAILED LEAK RATE TEST
			CORRECTIVE ACTION: REPLACED BONNET GASKET; LAPPED AND BLUE CHECKED
940829	MD	1	#12 AUXILIARY BUILDING SUPPLY FAN
			FAILURE DESCRIPTION: HEATING COILS RUPTURED
			CORRECTIVE ACTION: CRIMPED TUBING OF COILS AND SILVER BRAZED; TESTED WITH AIR AND SNOOPED
943631	MD	1	VALVE #12GB3
			FAILURE DESCRIPTION: VALVE FAILED LEAK RATE TEST
			CORRECTIVE ACTION: VALVE GASKET SURFACE MACHINED
0099100339	SMD	1	#11 CHILL WATER PUMP
			FAILURE DESCRIPTION: TRIPS ON OVERLOAD
			CORRECTIVE ACTION: INSTALLED NEW MOTOR; ECM LINED MOTOR TO PUMP

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-10-14-009-8	SMD	1	1CC131 LIMITORQUE
			FAILURE DESCRIPTION: VALVE FAILED TO MAKE OPEN LIMIT AND A BURNING SMELL WAS NOTED IN THE VICINITY OF THE BREAKER
			CORRECTIVE ACTION: REPLACED MOTOR ON LIMITORQUE; FIXED BROKEN ARM ON BREAK ASSEMBLY
009900281-7	SMD	1	#12 SERVICE WATER PUMP
			FAILURE DESCRIPTION: REPLACE AIR RELEASE VALVE AS PER DR #MD 84-3313
			CORRECTIVE ACTION: REPLACED VALVE AND TIGHTENED FITTINGS
84-09-29-007-3	SMD	1	#12 SERVICE WATER PUMP
			FAILURE DESCRIPTION: SEVERE PUMP PACKING LEAK
			CORRECTIVE ACTION: REPACKED PUMP AND STRAINER WITH BALZONA METAL KIT ON AIR RELEASE VALVE
84-09-12-004-6	SMD	1	#13 SERVICE WATER PUMP
			FAILURE DESCRIPTION: UPPER MOTOR BEARING OIL LEAK
			CORRECTIVE ACTION: REPLACED MOTOR
84-08-04-325-1	SMD	1	VALVE #11SW312 (SCREEN WASH DRAIN VALVE)
			FAILURE DESCRIPTION: VALVE IS ERODED AND LEAKS
			CORRECTIVE ACTION: WELDED IN NEW PIPE AND REPLACED VALVE

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
930189	NCS	1	VALVE 1SW26	
			FAILURE DESCRIPTION:	VALVE LEAKS
			CORRECTIVE ACTION:	REPLACED VALVE WITH NEW 30" PRATT BUTTERFLY VALVE
922465	NCS	1	VALVE 13SW20	
			FAILURE DESCRIPTION:	VALVE LEAKS
			CORRECTIVE ACTION:	REPAIRED RUBBER LINING AS PER ENGINEERING INSTRUCTIONS
0099128853	SMD	1	#11SW23 CONTROL ROOM INDICATOR	
			FAILURE DESCRIPTION:	LIMIT SWITCH STICKS WHEN OPENING VALVE
			CORRECTIVE ACTION:	CLEANED CONTACTS AND REPLACED 33Y-3 RELAY
009910958	SIC	1	VALVE 11MS10	
			FAILURE DESCRIPTION:	VALVE WON'T STAY IN AUTO
			CORRECTIVE ACTION:	REPLACED AUTO/MAN MODULE; CLEANED GEAR SHAFTS
0099128748	SIC	1	VALVE 11MS10	
			FAILURE DESCRIPTION:	VALVE WON'T OPEN
			CORRECTIVE ACTION:	CLEANED PLUGGED ORIFICE IN E/P

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-10-17-004-3	SMD	1	PRESSURIZER HEATERS
			FAILURE DESCRIPTION: DEIONS 4, 5, 6 ON BACKUP GROUP 11 (1GP3X) ARE TRIPPING OFF
			CORRECTIVE ACTION: REPLACED BREAKER
009900261-2	SMD	1	INNER CONTAINMENT DOOR EL. 100'
			FAILURE DESCRIPTION: REQUIRES REPAIR
			CORRECTIVE ACTION: FLANGE BLOCK BEARINGS TIGHTENED
84-08-10-649-0	SMD	1	100' EL. AIRLOCK
			FAILURE DESCRIPTION: OUTER SEAL DID NOT SEAL DURING CHECK
			CORRECTIVE ACTION: REPLACED WITH NEW SEALS AFTER CLEANING SEAL AREA
0099100771	SMD	1	#1 ALT SHUTDOWN SYSTEM INVERTER
			FAILURE DESCRIPTION: INVERTER DOES NOT WORK
			CORRECTIVE ACTION: REPLACED C2 CAPACITOR; OUTPUT TESTED
0099122146	SMD	1	REACTOR TRIP BREAKER SYSTEM
			FAILURE DESCRIPTION: BY-PASS BREAKER "A" FAILED TO CLOSE DURING P-4 TESTING
			CORRECTIVE ACTION: INSTALLED NEW SWITCH ON CONSOLE #GE-10CP264SBY

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
84-10-11-008-3	SMD	1	VALVE #13MS171	
			FAILURE DESCRIPTON:	STEAM LEAKING PAST SEAT
			CORRECTIVE ACTION:	BLUE CHECKED; REPLACED GASKETS (VALVE WAS MISSING SEAT RING GASKET)
84-05-10-216	SIC	1	VALVE 12MS169	
			FAILURE DESCRIPTION:	VALVE DIAPHRAGM REQUIRES REPLACEMENT
			CORRECTIVE ACTION:	INSTALLED NEW DIAPHRAGM AND CHECKED VALVE STROKE AND SEATING
84-05-219-1	SIC	1	VALVE #14MS169 (STOP VENT VALVE)	
			FAILURE DESCRIPTION:	VALVE DIAPHRAGM REQUIRES REPLACEMENT
			CORRECTIVE ACTION:	REPLACED DIAPHRAGM AND STROKED VALVE
0099130131	SMD	1	PR3, 4, AND 5	
			FAILURE DESCRIPTION:	OVERHEAD ALARM NOT FULLY SEATED (FLASHES)
			CORRECTIVE ACTION:	INSTALLED NEW CONTACT BLOCK AND CONTACT ARM; CHECKED CONNECTIONS; CHECKED TERMINATIONS; CALIBRATED
84-10-17-003-5	SMD	1	PRESSURIZER HEATERS	
			FAILURE DESCRIPTION:	DEIONS 7, 8, 9 ON 11 BACKUP GROUP HEATERS (1GP3X) ARE TRIPPING OFF
			CORRECTIVE ACTION:	REPLACED BREAKER

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-09-10-001-1	SMD	1	1A DIESEL GENERATOR AUTO BEZEL ALARM
			FAILURE DESCRIPTION: ALARM CANNOT BE ACKNOWLEDGED WHEN DIESEL IS TAKEN TO MANUAL CONTROL
			CORRECTIVE ACTION: REPLACED OPERATE/RESET COIL (STRUTHERS/DUNN 255XCX-P)
009910291	SMD	1	1C SEC
			FAILURE DESCRIPTION: TEST 18 WILL NOT RESET
			CORRECTIVE ACTION: REPLACED CARD ASSEMBLY WITH SPARE; REPLACED XK6 RELAY
009902271	SIC	1	NIS CH N-35
			FAILURE DESCRIPTION: READING OF 10^{-10} AMP WITH NO FUEL IN CORE
			CORRECTIVE ACTION: REPLACED DETECTOR
84-06-19-762-5	SIC	1	AUDIO COUNT RATE SCALER
			FAILURE DESCRIPTION: CHANNEL FAILED
			CORRECTIVE ACTION: REPLACED TIMER/SCALER WITH OLDER MODEL
0099128837	SIC	1	SOURCE RANGE CHANNEL I
			FAILURE DESCRIPTION: COUNTS WENT UP BY A FACTOR OF 10 WHILE CHANNEL II STAYED CONSTANT
			CORRECTIVE ACTION: REPLACED CONNECTOR ON CABLES; REPLACED PRE-AMP

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
OCTOBER 1984

Unit No. 1 began the period shutdown as the fifth refueling outage drew to a close. The Unit entered Mode 3 on 10/05/84 at 2013 hours. Repairs to a leaking pressurizer spray valve and a faulty source range channel delayed heatup and subsequent testing. On 10/13/84 at 1635 hours the Unit entered Mode 2 (reactor critical) for Low Power Physics Testing. On 10/14/84 at 1605 hours the reactor was shutdown to Mode 3 in accordance with Technical Specifications because of an inoperable Containment Isolation Valve 1CC131. Investigation revealed that the limitorque operator had failed and required replacement. On 10/14/84, in preparation for a reactor startup, testing revealed problems with one of the two manual reactor trip switches on the Control Room console which was subsequently replaced. On 10/15/84 at 2244 hours the Unit re-entered Mode 2 to continue Low Power Physics Testing which was completed on 10/16/84. On 10/16/84 No. 13 Condensate Pump lower motor bearing failed requiring replacement of the motor. As a result of problems with the Generator Seal Oil System during steady state roll at 1800 rpm, it was decided to inspect the generator. Investigation revealed the most likely cause was a failure of the 12 psig regulator and a plugged cunofilter on the discharge side of the Seal Oil Pumps. At 0317 hours on 10/21/84 the reactor was brought subcritical (Mode 3) due to three of four BF13 valves closing without cause. After extensive investigation, the valves retested satisfactorily and the reactor was brought critical on 10/21/84 at 2128 hours. The Unit was synchronized at 0343 hours on 10/22/84 and ran for a minimum of eight (8) hours prior to a scheduled Turbine Overspeed Test. At 1544, with the Unit load removed, the reactor tripped following completion of the Turbine Overspeed Test. The trip was caused by induced vibrations in the First Stage Pressure Transmitter (PT506) resulting in a false pressure signal which armed a reactor protection circuit (P-7). Following replacement and calibration of PT506, the reactor was brought critical and the Unit synchronized at 2207 hours on 10/23/84. The Unit was held at 84% power because of Condenser vacuum problems. On 10/29/84 the Unit was taken below P-8 (36% power) as a precautionary measure to prevent a reactor trip while repressurizing a reactor coolant pump flow transmitter instrument sensing line. Power ascension was resumed on 10/30/84 at 1107 hours. Due to continued high back pressure in the Condenser, the Unit was held at 45% power where it remained at the end of the period.

REFUELING INFORMATION

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: November 10, 1984
TELEPHONE: 609/935-6000
EXTENSION: 4455

COMPLETED BY: J. Ronafalvy

Month October 1984

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE 10/1/84

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

November 10, 1984

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of October 1984 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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