OPERATING DATA REPORT

DOCKET NO. 50-266

DATE November 7, 1984

COMPLETED BY C. W. KRAUSE

TELEPHONE 414 277 2001

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 1
2. REPORTING PERIOD: OCTOBER 1984
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.

NOTES

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
   NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745	7,320	122,616
12. NUMBER OF HOURS REACTOR WAS CRITICAL	745.0	4,956.1	99,034.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	4.3	629.7
14. HOURS GENERATOR ON LINE	745.0	4,916.0	96,523.5
15. UNIT RESERVE SHUTDOWN HOURS	0.0	9.0	802.5
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,114,168	7,217,132	130,752,444
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	384,780	2,491,490	43,887,470
18. NET ELECTRICAL ENERGY GENERATED (MWH)	368,508	2,376,888	41,739,322
19. UNIT SERVICE FACTOR	100.0	67.2	78.7
20. UNIT AVAILABILITY FACTOR	100.0	67.3	79.4
21. UNIT CAPACITY FACTOR (USING MDC NET)	102.0	67.0	69.6
22. UNIT CAPACITY FACTOR (USING DER NET)	99.5	65.3	68.5
23. UNIT FORCED OUTAGE RATE	0.0	0.0	2.5

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24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
    Annual refueling scheduled to commence April 19, 1985.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

DOCKET NO. 50-266
UNIT NAME Point Beach Unit 1
DATE November 7, 1984
COMPLETED BY C. W. Krause
TELEPHONE 414/277-2001

AVERAGE DAILY UNIT POWER LEVEL

MONTH October, 1984

DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET
1	501	11	500	21	503
2	500	12	500	22	502
3	502	13	500	23	500
4	500	14	502	24	502
5	314	15	500	25	502
6	500	16	498	26	502
7	500	17	502	27	502
8	501	18	495	28	502
9	500	19	500	29	500
10	500	20	502	30	502
				31	501

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1984

DOCKET NO. 50-266
UNIT NAME Point Beach Unit 1
DATE November 7, 1984
COMPLETED BY C. W. Krause
TELEPHONE 414/277-2001

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action To Prevent Recurrence
4	841005	S	0	H	4	N/A	ZZ	ZZZZZZ	Power reduction to maintain 345 KV system stable during Wisconsin Public Service Corporation line outage.

[1] F: Forced
S: Scheduled

[2] Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

[3] Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

[4] Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

[5] Exhibit I- Same Source

AD-28B
(01-78)

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No.: 50-266
Unit Name: Point Beach Unit 1
Date: November 7, 1984
Completed By: C. W. Krause
Telephone: 414/277-2001

Unit 1 operated at approximately 500 MWe net throughout the period with one major load reduction. On October 5, load was reduced to approximately 310 MWe net for 22 hours to maintain the 345 KV system stable during a Wisconsin Public Service Corporation line outage.

Primary-to-secondary leakage remains less than 10 gallons per day.

The return of the last spent fuel assembly occurred on October 14, 1984. This ends 14½ months of spent fuel shipments from facilities in New York and Illinois to the plant.

On October 18, 1984, the Unit 1 gas stripper released approximately 150 gallons of liquid via the recovery heat exchanger vent which was discovered uncapped. Air samples taken in the area indicated the release was below reportable limits.

On October 19, 1984, the power supplies for the Unit 1 containment air monitors were discovered full of water resultant from heavy rains leaking through the facade roof. I&C personnel replaced the power supplies and are preparing a modification to prevent recurrence. This event was not considered reportable.

On October 28, 1984, at 2:45 AM, control room operators received a ground fault alarm on motor control center 1B03 and a subsequent trip of 1P11A component cooling water pump. Redundant pump 1P11B automatically started to restore component cooling water system operation. Investigations indicate that insulation on the motor windings failed and caused a fire. The plant entered a 24-hour limiting condition for operation. Maintenance personnel replaced the failed motor with a motor from Unit 2 which was not needed and pump 1P11A returned to service and terminated the 24-hour limiting condition for operation.

Other safety-related maintenance completed during the period included the continuing work on the station battery and inverter modification, continuing work on the auxiliary safety instrumentation panel, and repair to auxiliary feedwater check valve AF-102.

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE November 7, 1984

COMPLETED BY C. W. KRAUSE

TELEPHONE 414 277 2001

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2
2. REPORTING PERIOD: OCTOBER 1984
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.

NOTES

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
   NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745	7,320	107,401
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	6,489.2	94,917.4
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	8.8	207.1
14. HOURS GENERATOR ON LINE	0.0	6,417.9	93,320.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	15.4	198.1
16. GROSS THERMAL ENERGY GENERATED (MWH)	0	9,542,695	130,437,472
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	3,229,550	44,189,380
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0	3,083,126	42,083,726
19. UNIT SERVICE FACTOR	0.0	87.7	86.9
20. UNIT AVAILABILITY FACTOR	0.0	87.9	87.1
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	86.8	79.8
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	84.7	78.8
23. UNIT FORCED OUTAGE RATE	0.0	0.0	1.3

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
    NONE

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOVEMBER 17, 1984

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

DOCKET NO. 50-301

UNIT NAME Point Beach Unit 2

DATE November 7, 1984

COMPLETED BY C. W. Krause

TELEPHONE 414/277-2001

AVERAGE DAILY UNIT POWER LEVEL

MONTH October, 1984

DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET
1	-5	11	-2	21	-2
2	-2	12	-2	22	-2
3	-2	13	-2	23	-2
4	-2	14	-2	24	-2
5	-2	15	-2	25	-2
6	-2	16	-2	26	-2
7	-2	17	-2	27	-2
8	-2	18	-2	28	-2
9	-2	19	-2	29	-2
10	-2	20	-2	30	-2
				31	-2

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1984

DOCKET NO. 50-301
UNIT NAME Point Beach Unit 2
DATE November 7, 1984
COMPLETED BY C. W. Krause
TELEPHONE 414/277-2001

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action To Prevent Recurrence
3	840928	S	745.0	C	1	N/A	ZZ	ZZZZZZ	Continuing 47-day refueling outage.

[1] F: Forced
S: Scheduled

[2] Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

[3] Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

[4] Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

[5] Exhibit I- Same Source

AD-28B
(01-78)

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No.: 50-301
Unit Name: Point Beach Unit 2
Date: November 7, 1984
Completed By: C. W. Krause
Telephone: 414/277-2001

Unit 2 was in a refueling shutdown during the entire period. Containment cleanup was completed on October 2, 1984.

During the "B" reactor coolant pump motor overhaul/inspection, Maintenance personnel discovered a rotor short in the stator windings and preparations are being made to change out the motor with the onsite spare.

Leak checks and eddy current inspections of the steam generators were completed during the period. Leak tests revealed one leaking tube in "A" steam generator. One leaking tube and one tube with a wet end were discovered in "B" steam generator. The eddy current inspection of "A" steam generator revealed 2 undefinable indications (squirrels), 3 tubes with indications <40% and 9 tubes with indications >40%. All but one indication were located within the tubesheet and all 14 tubes have been plugged. The eddy current inspection of "B" steam generator revealed one tube with a pluggable indication. This tube, along with the 2 tubes discovered during the leak check, were plugged.

While in preparation of unlatching control rods before lifting the reactor vessel head, it was discovered that several control rod drive shaft guide tube flexure heads were damaged. Entry into the cavity was made to remove the loose and broken flexure heads to eliminate any chance of the loose parts getting into the reactor coolant system. Thirty-seven damaged flexure heads were removed along with one 1/4" x 1/8" hex head bolt. A final video camera exam indicated only one additional damaged head. Subsequently, all flexure heads were removed and the inserts were replaced with flexureless inserts.

The Westinghouse inspection of the optimized fuel assemblies was completed with no indications or problems. The PaR exam of the reactor vessel was completed with no indications. All split pins were UT inspected. Three cracked split pins were discovered; both split pins on assembly B6 and one split pin on assembly B8. The cracks occurred in the flange-to-shank region and visual verification with UT results confirmed all nuts were present.

Other safety-related maintenance completed during the period included the replacement of source range detectors 2N31 and 2N32, the removal of the incore flux thimble tubes for flux thimble replacement, inspection and repairs to the main feed regulating

valves, main steam isolation valves and the main steam nonreturn valves, repairs to purge exhaust valve operator 2-3212, commencement of the installation of primary side loose parts monitoring system, replacement of the "A" residual heat removal pump, continuing work on the auxiliary safety instrumentation panel modification, pressurizer safety valve testing, safety-related hydraulic snubber surveillance, cleaning and eddy current testing of the "C" component cooling water heat exchanger, and the installation of a new fuel gripper assembly.

On October 22, 1984, the Unit 2 plant experienced the loss of normal AC power when a 13.8 KV bus tie was opened prior to the closure of the station transformer 2X03 low side breaker H06. Power was immediately restored by the 4D emergency Diesel. This event was reported to the NRC because of the safeguards actuation of the Diesel.

Weld repairs were completed on the moisture separator reheater steam channels and cladding is continuing on the turbine crossunder piping. Replacement of extraction steam piping is in progress. Also in progress is a modification to install a turbine bearing oil lift system.

Other secondary plant work included the cleaning and inspection of the turbine lube oil cooler tube bundle, steam turbine and generator overhaul, flushing and filtering of the turbine lube oil system, machining of the governor valve gasket seats, rebabbiting of 3 turbine low pressure rotor bearings, and the cleaning and eddy current testing of the generator hydrogen coolers, both trains of the 5 feedwater heaters, the condensate cooler, and the 4 condenser waterboxes. Further tube degradation was noted in the waterboxes. Examinations revealed the indications to be the result of ID tube inlet and outlet erosion and pitting, OD ammonia grooving near the air ejector penetration, and OD tube steam erosion over the top of the tube bundles.

Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

November 9, 1984

Director of Regulatory Operations
U. S. NUCLEAR REGULATORY COMMISSION
Washington, D. C. 20555

Gentlemen:

MONTHLY OPERATING REPORTS
POINT BEACH NUCLEAR PLANT

Attached are monthly operating reports for Units 1 and 2, Point Beach Nuclear Plant, for the calendar month of October 1984.

Very truly yours,

C. W. Fay

Vice President-Nuclear Power

C. W. Fay

Attachments

Copies to J. G. Keppler - NRC, Region III
NRC Resident Inspector
R. S. Cullen - PSCW

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