

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3

DOCKET NUMBER (2)

0 5 0 0 0 3 6 2

PAGE (3)

1 OF 0 4

TITLE (4)

DELINQUENT RCS SAMPLE WITH DOSE EQUIVALENT IODINE LIMITS EXCEEDED

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	3 0	8 5	8 5	0 0 4	0 1 0	0 1	2 2	8 5		0 5 0 0 0 1 1
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										
OPERATING MODE (9)		5	20.402(b)			20.405(c)			50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		0 1 0 0	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)	73.71(c)
			20.405(a)(1)(ii)		X	50.36(c)(2)			50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iii)		X	50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME
J. G. HAYNES, STATION MANAGER

TELEPHONE NUMBER

AREA CODE

7 1 4 4 9 2 - 1 7 7 0 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NO ☐

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 30, 1985, at 0900, with Unit 3 in Mode 5, following collapsing of the pressurizer steam bubble after a shutdown, analysis of a Reactor Coolant System (RCS) sample, taken only for observation purposes and not required by Technical Specifications, indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131.

RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by 1930 with purification flow. The increased activity was caused when the iodine in the pressurizer steam bubble was forced into solution while collapsing the bubble.

A 4 hour sample due to be taken at 1650 was not taken until 1930. Our investigation of the administrative controls surrounding the delinquent sample determined that the cause of the delinquent sample was personnel error in that the responsible chemistry technician and chemistry foreman understood the sampling requirements but failed to complete them within the required time. Appropriate disciplinary action was taken. Additionally, this event was discussed with all chemistry technicians and foremen.

This submittal also provides the report pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd', RCS specific activity exceeding 1.0 microcuries/gram Dose Equivalent I-131.

8503130463 850227
PDR ADOCK 05000362
S PDRIE 22
11

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5 0 0 0 3 6 2	LER NUMBER (6)			PAGE (3)	
		YEAR 8 5	SEQ. NUMBER - 0 0 4	REV. NUMBER - 0 0	0 2	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On January 30, 1985, at 0900, with Unit 3 in Mode 5, following collapsing of the pressurizer steam bubble after a shutdown, analysis of a Reactor Coolant System (RCS)(EIIS System Code AB) sample, taken only for observation purposes and was not required by Technical Specifications, indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131.

RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by 1930 with purification flow. The increased activity was caused when the iodine in the pressurizer steam bubble was forced into solution while collapsing the bubble.

A 4 hour sample due to be taken at 1650 was not taken until 1930. Our investigation of the administrative controls surrounding the delinquent sample determined that the cause of the delinquent sample was personnel error in that the responsible chemistry technician and chemistry foreman understood the sampling requirements but failed to complete them within the required time. Appropriate disciplinary action was taken. Additionally, this event was discussed with all chemistry technicians and foremen.

This submittal also provides the report pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd', Reactor Coolant System (RCS) specific activity exceeding 1.0 microcuries/gram Dose Equivalent I-131.

Additional information, required by LCO 3.4.7, Action Statement 'd', is provided on the following pages. Although the unit has a degassification path which operates continuously and takes pressurizer steam, condenses it, and directs it to Liquid Radwaste, degassing operation history is not applicable, because this system reduces the noble gas content of the Reactor Coolant System but has no effect on iodine.

CLEANUP FLOW HISTORYPERIODAVERAGE CLEANUP
FLOW (GPM)

1/28/85, 0900 to 1/30/85, 1930

85*

*Hourly cleanup flow data not available. Figure used is taken from average flow with two charging pumps in operation.

REACTOR POWER HISTORYPERIODREACTOR POWER

1/28/85, 0900 to 1/30/85, 1930

0%

REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY ANALYSISDATETIMEDE I-131
MICROCURIES/GRAM

1/30/85

0900

1.41

1/30/85

1250

1.36

1/30/85

1930

0.85

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (5)

PAGE (3)

SAN ONOFRE NUCLEAR GENERATING STATION,
UNIT 3

0 5 0 0 0 3 6 2 8 5 1 0 0 4 1 0 0 0 3 OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PAGE =

09:43:35

01/21/85

EDIT= 23

AXIALLY INTEGRATED AND PEAK OUTPUT ASSEMBLY EXPOSURE EDITS

53C1F 032

M2 1280

FORMAT OF ASSEMBLY IN CORE MAP
ASSEMBLY NUMBER - BATCH NUMBER
INTEGRATED BOX EXPOSURE IN 10⁻⁰⁰³MMWD/T
MAXIMUM BOX EXPOSURE IN 10⁻⁰⁰³MMWD/T
LOCATION OF MAX. ASS. EXP. IN C/O HEIGHT1-05 2-05 3-05 4-05
5-580 7-253 7-248 5-567
6-580 8-919 8-904 6-854
34-000 34-000 34-000 34-0005-05 6-05 7-05 8-05 9-05 10-05 11-05 12-05 13-05
5-376 7-258 8-580 10-261 9-026 10-243 8-600 7-181 5-240
6-722 8-969 10-320 12-783 11-406 12-740 10-728 8-303 6-466
35-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 42-00014-05 15-05 16-05 17-05 18-05 19-05 20-05 21-05 22-05 23-05 24-05
5-871 8-885 9-004 10-206 9-572 10-048 9-183 8-933 7-771 5-803
7-222 11-209 11-561 12-528 11-837 12-497 11-299 10-810 7-087
35-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-00025-05 26-05 27-05 28-05 29-05 30-05 31-05 32-05 33-05 34-05 35-05
5-952 7-331 9-073 10-295 10-000 10-754 9-851 10-227 9-450 8-975
7-355 11-311 11-451 12-596 12-217 13-267 12-434 11-283 10-176 9-128 37-05
36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 34-00036-05 37-05 38-05 39-05 40-05 41-05 42-05 43-05 44-05 45-05 46-05 47-05 48-05 49-05 50-05 51-05 52-05
5-217 8-850 9-161 10-348 10-000 10-754 9-851 10-227 9-450 8-975 51-36 52-05
6-356 10-398 11-330 12-759 12-217 13-267 12-434 11-283 10-176 9-128 5-163
36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-000 36-00053-05 54-05 55-05 56-05 57-05 58-05 59-05 60-05 61-05 62-05 63-05 64-05 65-05 66-05 67-05 68-05 69-05 70-05
5-05 8-836 9-308 10-398 10-056 10-851 10-227 9-450 8-975 51-36 52-05
7-255 10-398 11-330 12-759 12-217 13-267 12-434 11-283 10-176 9-128 5-163
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68-07 69-02 70-04 71-02 72-03 73-01 74-03 75-01 76-03 77-01 78-03 79-01 80-03 81-01 82-01 83-01 84-01 85-01
11-065 11-345 12-694 12-258 13-382 12-603 13-608 12-706 13-577 12-350 13-300 12-161 10-222 10-222 10-222 10-222 10-222
34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-00089-05 90-05 91-05 92-05 93-05 94-05 95-05 96-05 97-05 98-05 99-05 100-05 101-05 102-05 103-05 104-05 105-05 106-05
85-07 86-04 87-02 88-04 89-01 90-03 91-01 92-03 93-01 94-03 95-01 96-04 97-02 98-04 99-07 100-04 101-04 102-04
10-522 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275
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102-04 103-02 104-04 105-02 106-03 107-01 108-03 109-01 110-03 111-01 112-03 113-02 114-04 115-02 116-04 117-07 118-07 119-07
10-522 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275
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109-07 110-05 111-03 112-05 113-02 114-04 115-02 116-04 117-07 118-07 119-07 120-05 121-02 122-05 123-05 124-05 125-05 126-05
10-522 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275
34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000143-05 144-05 145-05 146-05 147-05 148-05 149-05 150-05 151-05 152-05 153-05 154-05 155-05 156-05 157-05 158-05 159-05 160-05
136-07 137-02 138-04 139-02 140-03 141-01 142-03 143-01 144-03 145-01 146-03 147-02 148-04 149-02 150-07 151-07 152-07 153-07
10-522 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275 10-275
34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000 34-000

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1)

SAN ONOFRE NUCLEAR GENERATING STATION,
UNIT 3

DOCKET NUMBER (2)

05000362815-0014-0004 OF 014

LER NUMBER (6)

PAGE (3)

TEXT (If more space is required, use additional NRC Form 366A's) (7)

32-000 9.032 9.270 13.254 9.933 10.873 10.386 11.174 10.521 11.144 10.535 10.841 9.944 10.228 9.218 8.945 32-
11.136 11.423 12.561 12.135 13.334 12.541 13.532 12.798 13.590 12.572 13.264 12.079 12.548 11.283 11.004
34.000 34.000 34.000 36.000 36.000 36.000 36.000 36.000 36.000 36.000 36.000 36.000 36.000 34.000 34.000

151-05 152-04 153-02 154-04 155-02 156-03 157-31 158-33 159-31 160-33 161-02 162-04 163-02 164-04 165-05
7.248 3.327 9.278 12.324 10.051 10.861 11.356 11.331 10.311 10.740 9.375 10.225 9.238 8.351 7.221
8.915 11.014 11.403 12.690 12.259 13.316 12.574 13.392 12.526 13.200 12.130 12.434 11.255 10.923 8.803
34.000 34.000 35.000 35.000 36.000 36.000 35.000 34.000 36.000 36.000 36.000 38.000 36.000 34.000 34.000

166-05 167-06 168-04 169-02 170-04 171-02 172-04 173-02 174-04 175-02 176-04 177-02 178-04 179-06 180-05
5.209 8.832 9.139 9.296 10.348 10.024 10.796 10.107 10.748 9.940 10.270 9.234 9.067 8.750 5.175
6.431 10.920 11.329 11.388 12.732 12.237 13.268 12.411 13.211 12.136 12.606 11.255 11.133 10.708 6.298
34.000 35.000 35.000 35.000 36.000 36.000 34.000 34.000 36.000 36.000 35.000 38.000 36.000 34.000 34.000

181-05 182-02 183-04 184-02 185-04 186-02 187-04 188-02 189-04 190-02 191-04 192-02 193-05
5.970 7.316 9.032 9.295 10.260 9.901 10.514 9.857 11.139 9.234 9.014 7.435 5.782
7.314 9.255 11.151 11.387 12.639 12.121 12.968 12.068 12.481 11.305 11.139 9.058 6.998
34.000 35.000 38.000 36.000 34.000 34.000 36.000 36.000 38.000 38.000 38.000 36.000 34.000

194-05 195-06 196-04 197-02 198-04 199-02 200-04 201-02 202-04 203-06 204-05
5.852 8.846 8.984 9.208 10.080 9.602 10.058 9.170 8.939 8.800 5.808
7.139 10.881 11.087 11.283 12.463 11.795 12.444 11.245 11.030 10.810 7.063
35.000 36.000 36.000 34.000 36.000 36.000 36.000 36.000 36.000 36.000 36.000

205-05 206-05 207-07 208-07 209-04 210-07 211-07 212-05 213-05
5.327 7.221 8.629 10.257 9.143 10.270 8.652 7.201 5.285
6.534 8.821 10.616 12.659 11.331 12.680 10.591 8.802 6.464
34.000 34.000 34.000 34.000 34.000 34.000 34.000 34.000 34.000

214-05 215-05 216-05 217-05
5.574 7.268 7.269 5.580
6.805 8.846 8.854 6.826
34.000 34.000 34.000 34.000

MAXIMUM INTEGRATED ASSEMBLY EXPOSURE IS 0.112445D+05 MWD/T IN ASSEMBLY 110
MAXIMUM PEAK AXIAL EXPOSURE IS 0.137041D+05 MWD/T, OCCURRING AT 36.00 O/D OF THE CORE HEIGHT IN ASSEMBLY 108
CORE AVERAGE EXPOSURE IS 3.923558D+04 MWD/T
Equal to 243.42 EFPD

----- BATCH AVERAGE EXPOSURES -----

BATCH NUMBER	BATCH NAME	AVERAGE EXPOSURE (GWD/T)
1	A1	10.395
2	A2	9.468
3	B1	11.007
4	B2	9.879
5	C	6.282
6	C+	8.811
7	C+	9.611

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

February 27, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 85-004
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.36(c)(2), 10 CFR 50.73(a)(2)(i), and Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Coolant System specific activity. Neither the health and safety of plant personnel nor the public were affected by this event.

If you require any additional information, please so advise.

Sincerely,

JG Haynes

Enclosure: LER No. 85-004

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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