



Commonwealth Edison

Quad Cities Nuclear Power Station
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RLB-92-166

July 24, 1992

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Reference: Quad Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One

Enclosed is Licensee Event Report (LER) 92-016, Revision 00, for Quad Cities Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(i)(B). The licensee shall report any operation or condition prohibited by the plant's Technical Specification.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD CITIES NUCLEAR POWER STATION

R. L. Bax

R. L. Bax
Station Manager

RLB/TB/rjb

Enclosure

cc: J. Schrage
R. Higgins
INPO Records Center
NRC Region III

STMGR 411

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PDR ADDCK 05000254
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LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) Quad Cities Unit One
 Title (4)
 Docket Number (2) 0 | 5 | 0 | 0 | 0 | 2 | 5 | 4
 Page (3) 1 | of | 0 | 4

Missed Tech Spec Surveillance Due To Personnel Error

Event Date (5)			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0 6	2 6	9 2	9 2	0 1 6	0 0	0 7	2 4	9 2		0 5 0 0 0 1 1

OPERATING MODE (9) 4
 POWER LEVEL (10) 0 | 9 | 2 %
 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)
 20.402(b) 20.405(c) 50.73(a)(2)(iv) 73.71(b)
 20.405(a)(1)(i) 50.36(c)(1) 50.73(a)(2)(v) 73.71(c)
 20.405(a)(1)(ii) 50.36(c)(2) 50.73(a)(2)(vii) Other (Specify
 20.405(a)(1)(iii) X 50.73(a)(2)(i) 50.73(a)(2)(viii)(A) in Abstract
 20.405(a)(1)(iv) 50.73(a)(2)(ii) 50.73(a)(2)(viii)(B) below and in
 20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(x) Text)

LICENSEE CONTACT FOR THIS LER (12)

Name J. M. Masterlark, Tech Staff Engineer Ext. 2619
 TELEPHONE NUMBER
 AREA CODE 3 | 0 | 9 | 6 | 5 | 4 | - | 2 | 2 | 4 | 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

Expected Submission Date (15) Month | Day | Year
 Yes (If yes, complete EXPECTED SUBMISSION DATE) X | NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT:

At 1015 hours on June 26, 1992, Unit One was in the RUN mode at 97% rated core thermal power. At this time, the Fire Marshal discovered that valve 1-4199-72 had not been verified "locked open" within the Technical Specification surveillance interval. It was determined that Technical Specification 4.12 had not been met.

Personnel who reviewed QOS 4100-S3 failed to notice that the valve had not been verified in the "locked open" position. Immediate corrective action was to verify that valve 1-4199-72 was in the "locked open" position. A memo will be sent to all personnel who review Technical Specification Surveillances on the importance of verifying that the surveillances are complete. A procedure change will also be submitted to allow for verification of hard to reach valves with a flow test downstream of the valve.

This report is being submitted in accordance with the requirements of 10CFR50.73(a)(2)(i).

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							Year	Sequential Number	Revision Number																
Quad Cities Unit One			0	5	0	0	0	2	5	4	9	2	-	0	1	6	-	0	1	0	0	2	Of	0	4

Energy Industry Identification System (EIIS) codes are identified in the text as [XX]

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power.

EVENT IDENTIFICATION: Missed Tech Spec Surveillance Due To Personnel Error.

A. CONDITIONS PRIOR TO EVENT:

Unit: One Event Date: June 26, 1992 Event Time: 1015
Reactor Mode: 4 Mode Name: Run Power Level: 92%

This report was initiated by Deviation Report D-4-1-92-068

RUN Mode (4) - In this position the reactor system pressure is at or above 825 psig, and the reactor protection system is energized, with APRM protection and RBM interlocks in service (excluding the 15% high flux scram).

B. DESCRIPTION OF EVENT:

At 1015 hours on June 26, 1992, Quad Cities Unit One was in the RUN mode at 97% rated core thermal power. During a review of Surveillance QOS 4100-S3 Monthly Fire Suppression Valve Position (performed on March 10, 1992), the Fire Marshall discovered that a valve 1-4199-72 had not been verified in the "locked-open" position. "Inaccessible" was entered on the surveillance data sheet for that valve. It could not be determined who wrote this on the data sheet. The valve is located in a hard to access location.

Per Technical Specification 4.12.B.1.b, the Fire Suppression Water System valve line-up shall be verified every 31 days. The surveillance was last performed on February 8, 1992 and was subsequently performed during March 9, and March 10, 1992. During review of the procedure, it was found that valve 1-4199-72 was not verified in a "locked open" position within the 31 day requirement.

It was concluded that Technical Specification 4.12.B.1.b had not been met for Unit 1. QOS 4100-S3 has been performed twice since, April 10, 1992, and May 11, 1992. Both times that valve has been verified in a "locked open" position.

C. APPARENT CAUSE OF EVENT:

This report is being submitted in accordance with the requirements of 10CFR50.73(a)(2)(i) which requires that the licensee report any operation or condition prohibited by the plant's Technical Specifications.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		Year	///	Sequential Number	///	Revision Number			
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TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

The cause of this event is personnel error. The procedure was to be completed once every 31 days. The procedure was performed, however it was not determined until after the 31 days, during the review process that one of the valves had been overlooked.

Personnel who reviewed QOS 4100-S3 failed to notice that the valve had not been verified in the "locked-open" position within the allotted time frame.

D. SAFETY ANALYSIS OF EVENT:

The safety of plant and personnel was not affected by this event. The valve position had been verified within 31 days before March 10, 1992, and within 31 days after March 10, 1992. Both times the valve had been verified in the "locked open" position, and the valve would not have been manipulated during that period of time due to its location. The system would have been operable in the event of a fire.

E. CORRECTIVE ACTIONS:

Immediate corrective action was to review preceding surveillances to verify that valve 1-4199-72 had been verified "locked open".

A memo will be sent to all personnel who review Technical Specification Surveillances on the importance of verifying that the surveillances are complete (NTS #2542009206801).

The Tech Staff System engineer will modify QOS 4100-4 "Monthly Fire Suppression Valve Position Inspection" to allow position verification of hard to reach valves with a flow test downstream of the valve (NTS #2542009206802).

F. PREVIOUS EVENTS:

The following LERs have resulted from missed Technical Specification surveillances due to personnel error:

LER	DESCRIPTION
1-88-015	Missed Tech Spec Fire Surveillance
1-89-013	Missed Two Four-Hour Surveillances With Main Chimney Sample Alarm Out of Service
1-90-014	Missed RETS Sample Following Reactor Power Increase

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TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]

LER	DESCRIPTION
1-90-019	Missed Tech Spec Surveillance on the MSL Rad Monitors
1-90-024	Missed Tech Spec Surveillance for Continuous Fire Watch
1-90-027	Improperly Performed T.S. Surveillance While in EGC
1-91-020	Missed Tech Spec Surveillance for Recombiner Outlet Temperature/RX Power
2-91-013	Entered EGC Without Performing the Required Surveillances
1-92-011	"A" Recirc Loop Temp Instrumentation INOP (Missed Tech Spec Surveillance)

G. COMPONENT FAILURE DATA:

No component failures were involved with this event.