

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 8 1 3										PAGE (3) 1 OF 0 2																					
TITLE (4) Inadvertent Control Room Ventilation Isolation																																									
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																							
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)								
0 1			2 5			8 5			8 5			0 0			8			0 0			0 2			2 5			8 5									0 5 0 0 0					
0 1			2 5			8 5			8 5			0 0			8			0 0			0 2			2 5			8 5									0 5 0 0 0					
OPERATING MODE (9) 1						THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																			
POWER LEVEL (10) 1 0 0						20.402(b)						20.405(c)						<input checked="" type="checkbox"/> 50.73(a)(2)(iv)						73.71(b)																	
						20.405(a)(1)(i)						50.36(c)(1)						50.73(a)(2)(v)						73.71(c)																	
						20.405(a)(1)(ii)						50.36(c)(2)						50.73(a)(2)(vii)						OTHER (Specify in Abstract below and in Text, NRC Form 366A)																	
						20.405(a)(1)(iii)						50.73(a)(2)(i)						50.73(a)(2)(viii)(A)																							
						20.405(a)(1)(iv)						50.73(a)(2)(ii)						50.73(a)(2)(viii)(B)																							
20.405(a)(1)(v)						50.73(a)(2)(iii)						50.73(a)(2)(ix)																													
LICENSEE CONTACT FOR THIS LER (12)																																									
NAME Charles D. Naslund - Superintendent, I&C																TELEPHONE NUMBER AREA CODE 3 1 4 6 7 6 - 8 5 1 0 0																									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																									
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																							
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR																			
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)																<input checked="" type="checkbox"/> NO																									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 1/25/85 a Containment Purge Isolation and Control Room Ventilation Isolation occurred. All Engineered Safety Features equipment functioned as designed.

Prior to this event, the Pressurizer Power Operated Relief Valve block valve was being stroke tested. The containment was not being purged at the time of this event. It is suspected that a small increase in gaseous activity that occurred when the block valve was stroked had caused a containment process monitor to exceed its alarm setpoint. No equipment malfunctions were discovered.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time did this event pose a threat to the public health or safety.

8503070179 850225  
PDR ADOCK 05000483  
S PDR

IE 22  
Y

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Callaway Plant Unit 1

0 5 0 0 0 4 8 3 8 5 - 0 0 8 - 0 0 0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 385A's) (17)

On 1/25/85 at 1347 CST, a Containment Purge Isolation Signal (CPIS) and Control Room Ventilation Isolation Signal (CRVIS) were actuated due to a spurious signal on containment process monitor GT-RE-31.

Prior to this event, surveillance procedure OSP-BB-V0001, "SECTION XI RCS VALVE OPERABILITY," was in progress and a Pressurizer PORV block valve was being stroke tested. The containment was not being purged at the time of this event. Additionally, monitor GT-RE-31 was operating in the alert mode, with the alert setpoint at  $4.0\text{E-}4$  uC/ml, prior to the event. It is suspected that as the PORV block valve was stroked open, a small increase in gaseous activity caused the GT-RE-31 reading to exceed the alarm setpoint of  $5.95\text{E-}4$  uC/ml, and a CPIS/CRVIS was actuated.

Subsequent investigation revealed no equipment malfunctions or abnormalities from containment process monitors GT-RE-31 or GT-RE-32. A small upward trend in containment activity was observed to have taken place at the time of the stroke test. However, a high activity alarm did not occur on the associated alarm recorders. The reason no high activity alarm was received is that the ESF actuations are initiated by the analog output section of the process monitor, while the alarms are generated by converted digital signals. A  $\pm 2.5\%$  tolerance is associated with these two signals, and due to the small size of the activity increase, the actuations occurred on the low tolerance value and no alarm was generated.

Containment process monitors GT-RE-31 and GT-RE-32 were placed in bypass as they are not required to be in service providing that no containment purge is in progress and all containment purge valves are closed.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time did this event pose a threat to the public health or safety.

Previous occurrences: none

UNION ELECTRIC COMPANY  
CALLAWAY PLANT

MAILING ADDRESS:  
P.O. BOX 620  
FULTON, MO. 65251

February 25, 1985

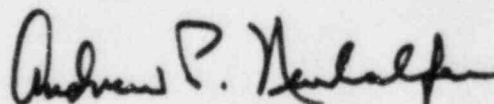
U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

ULNRC-1044

Gentlemen:

DOCKET NUMBER 50-483  
CALLAWAY PLANT UNIT 1  
FACILITY OPERATING LICENSE NPF-30  
LICENSEE EVENT REPORT 85-008-00  
INADVERTENT CONTROL ROOM VENTILATION ISOLATION

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning an inadvertent Control Room Ventilation Isolation Signal caused by a spurious radiation monitor signal.



for

S. E. Miltenberger  
Manager, Callaway Plant

CDN/JMS/drs  
Enclosure

cc: Distribution attached

FE22  
1/1

cc distribution for ULNRC-1044

Mr. James G. Keppler  
Regional Administrator  
Office of Inspection & Enforcement  
U.S. Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

American Nuclear Insurers  
c/o Dottie Sherman, Library  
The Exchange Suite 245  
270 Farmington Avenue  
Farmington, CT 06032

Records Center  
Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, GA 30339

NRC Resident Inspector  
Missouri Public Service Commission  
D. F. Schnell  
J. F. McLaughlin  
J. E. Davis (Z40LER)  
D. W. Capone/R. P. Wendling  
F. D. Field  
R. L. Powers  
A. C. Passwater/D. E. Shafer/D. J. Walker  
G. A. Hughes  
W. R. Robinson (QA Record)  
C. D. Naslund  
J. M. Price  
R. A. McAleenan  
L. K. Robertson (470)(NSRB)  
Merlin Williams, Wolf Creek  
SEM Chrono  
3456-0021.6  
3456-0260  
Z40ULNRC  
G56.37  
N. Date