

LICENSEE EVENT REPORT (LER)

APPROVED OMS NO. 2180 0104
EXPIRES - 8/31/95FACILITY NAME (1)
Limerick Generating Station - Unit 1DOCKET NUMBER (2)
0 5 0 0 0 3 5 2 1 OF 0 3

TITLE (4)

Reactor Enclosure Ventilation System Isolations

EVENT DATE (6)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER (8)																																																					
0	12	6	8	5	8	5	0	1	8	00	0	2	25	8	5			0	5	0	0	0																																										
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																																													
2			<table border="0"><tr><td>20.402(a)</td><td></td><td>20.406(a)</td><td><input checked="" type="checkbox"/></td><td>60.73(a)(2)(iv)</td><td></td><td>73.71(a)</td></tr><tr><td>POWER LEVEL (10)</td><td>0</td><td>0</td><td>4</td><td>20.406(a)(1)(ii)</td><td></td><td>60.73(a)(2)(v)</td><td></td><td>73.71(a)</td><td colspan="4" rowspan="5">OTHER (Specify in Abstract below and in Text, NRC Form 204A)</td></tr><tr><td></td><td></td><td></td><td>20.406(a)(1)(iv)</td><td></td><td>60.73(a)(2)(vi)</td><td></td></tr><tr><td></td><td></td><td></td><td>20.406(a)(1)(iii)</td><td></td><td>60.73(a)(2)(vii)</td><td></td></tr><tr><td></td><td></td><td></td><td>20.406(a)(1)(v)</td><td></td><td>60.73(a)(2)(viii)</td><td></td></tr><tr><td></td><td></td><td></td><td>20.406(a)(1)(vi)</td><td></td><td>60.73(a)(2)(ix)</td><td></td></tr></table>														20.402(a)		20.406(a)	<input checked="" type="checkbox"/>	60.73(a)(2)(iv)		73.71(a)	POWER LEVEL (10)	0	0	4	20.406(a)(1)(ii)		60.73(a)(2)(v)		73.71(a)	OTHER (Specify in Abstract below and in Text, NRC Form 204A)							20.406(a)(1)(iv)		60.73(a)(2)(vi)					20.406(a)(1)(iii)		60.73(a)(2)(vii)					20.406(a)(1)(v)		60.73(a)(2)(viii)					20.406(a)(1)(vi)		60.73(a)(2)(ix)	
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LICENSEE CONTACT FOR THIS LER (12)
NAME
John C. Nagle, Engineer - Special ProjectsTELEPHONE NUMBER
AREA CODE
2 1 5 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NOEXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 85-018

On January 26, 1985, the 'A' reactor enclosure ventilation isolation system, the reactor enclosure recirculation system and the standby gas treatment system initiated upon receiving an inadvertent reactor enclosure low differential pressure signal. Immediately after the isolation, the low differential pressure signal was reset and the affected systems were returned to normal operation. Approximately 30 minutes after the event, a second similar isolation occurred. Cause of both isolations was failure to isolate a refuel floor differential pressure transmitter from a reactor enclosure differential pressure transmitter (the two transmitters share a common reference leg) prior to troubleshooting the refuel floor instrument loop.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2 8 5	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	0 1 1 8	0 0	0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 368A (17))

Description of the Event:

On January 26, 1985, at 4:00 a.m., the 'A' reactor enclosure ventilation isolation system, the reactor enclosure recirculation system, and the standby gas treatment system initiated upon receiving an inadvertent reactor enclosure low differential pressure signal. At 4:33 a.m., on the same date, these systems received a second similar initiation signal. Immediately after each initiation, the low differential pressure signal was reset and all affected systems were returned to normal operation.

Consequences of the Event:

The 'A' reactor enclosure ventilation isolation system, the reactor enclosure recirculation system, and the standby gas treatment system operated properly during each initiation. There were no adverse consequences.

Cause of the Event:

At the time of the event, instrument and controls technicians were troubleshooting a downscale condition on refuel floor differential pressure indicator PDI-76-099A. While attempting to obtain a response on this indicator, the technicians opened the equalizer valve on the transmitter which provides the input signal to PDI-76-099A. The equalizer valve on this transmitter, PDT-76-399A, was opened without isolating the transmitter from the process.

The technicians did not realize that refuel floor differential pressure transmitter PDT-76-399A shares a common outside-air reference leg with reactor enclosure differential pressure transmitter PDT-76-498A. When the equalizer valve was opened on PDT-76-399A, the differential pressure sensed by both transmitters (PDT-76-399A and PDT-76-498A) was reduced to a value that initiated the reactor enclosure isolation systems.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 366A s) (17)

While operators were attempting to identify the cause of the first isolation, a second similar isolation occurred for the same reason.

The operators were aware that technicians were troubleshooting a refuel floor differential pressure instrument loop; however, they were not immediately aware of the interconnection between the refuel floor and reactor enclosure differential pressure transmitters. Therefore, the technicians were not requested to stop their work after the first isolation. After the second isolation, the operators determined that the work being performed by the technicians was the cause of the isolation and requested the technicians to stop their work on the refuel floor instrument loop.

Corrective Actions:

All of the instrument and controls technicians are currently attending an instrument valving training program.

Research and testing procedure RT-11-00467 (procedure for filling, venting and valving instruments) has been drafted and is in the process of being approved.

The technicians involved with this event were counseled as to the consequences of their actions and the importance of observing proper valving techniques.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

February 25, 1985

Docket No. 50-352

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Washington, DC 20555

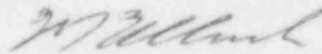
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER concerns the isolations of the reactor enclosure ventilation system.

Reference:	Docket No. 50-352
Report Number:	85-018
Revision Number:	00
Event Date:	January 26, 1985
Report Date:	February 25, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

IF22
1/1

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Atomic Safety & Licensing Board Panel
Docket & Service Section (3 Copies)
James Wiggins
Timothy R. S. Campbell

January 16, 1985