NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Washington Nuclear Project - Unit 2	0 5 0 0 0 3 9 7	1 OF 0 3

TITLE (4)
Isolation Actuation Instrumentation (Temperature Monitors)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
*			8 4	— 0 3 3	— 0 2	0 9	1 7	8 4		0 5 0 0 0

OPERATING MODE (9)	*	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	*	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	50.72(b)(2)(ii)
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
R. L. Koenigs, Compliance Engineer	5 0 9 3 7 7 - 2 5 0 1 Ext. 2279

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	I J	- T I S	R 2 7 9	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO		1 2	0 1	8 4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During plant heatup isolations have occurred due to conservative initial setpoints of the Temperature Monitoring Switches which actuate the Nuclear Steam Supply Shutoff System (NSSSS). One isolation occurred during reactor operations at 65% power and another at 45% power.

Actuations on 4-12-84, 4-18-84, 4-19-84, 8/14/84, 8/31/84 and 9/5/84 caused isolation of the Reactor Water Cleanup (RWCU) system. The area of the alarm was inspected to verify there was no steam leakage. New setpoints were determined, the switches reset, and the RWCU system returned to service.

*Event 1 - 4/12/84, Operating Mode - 2, Power Level - 001
Event 2 - 4/18/84, Operating Mode - 2, Power Level - 001
Event 3 - 4/19/84, Operating Mode - 2, Power Level - 001
Event 4 - 8/14/84, Operating Mode - 2, Power Level - 001
Event 5 - 8/31/84, Operating Mode - 1, Power Level - 065
Event 6 - 9/5/84, Operating Mode - 1, Power Level - 045

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Operating Conditions Prior to the Events:

	Event 1	Event 2	Event 3	Event 4	Event 5	Event 6
a) Power Level	1%	1%	1%	1%	65%	45%
b) Operational Mode	2	2	2	2	1	1

During initial Plant heatup there have been four Nuclear Steam Supply Shutoff System (NSSSS) isolations of the Reactor Water Cleanup System (RWCU). One isolation occurred with the reactor at power operation conditions. The isolations were caused by the Leak Detection Temperature Monitors. Table 3.3.2-2 of the Technical Specifications is footnoted to point out that the Leak Detection Temperature Switch setpoints will be determined during the Startup Test Program. The present setpoints are set conservatively low and are adjusted to higher values when the existing setpoint is approached or reached.

Prior to increasing the setpoint an inspection of the area is made to insure temperature increases are not due to leaks. It is anticipated that other isolations may occur if a temperature setpoint is reached prior to increasing the setpoint as Plant heat load increases during the Startup Test Program.

On 4-12-84 a Division II Leak Detection Temperature Switch monitoring the RWCU pipe routing area actuated and initiated an NSSSS isolation signal which closed RWCU-V-1, causing the subsequent loss of the RWCU system. The temperature switch actuation was due to a conservative setting of the trip setpoint. An inspection of the pipe routing area found that there were no leaks and the temperature switch was adjusted to a higher value, which was still below the allowable value listed in Table 3.3.2-2 of the Technical Specifications. Per Technical Specification 3.3.2 the following note to Table 3.3.2-2 is quoted concerning listed setpoint values.

NOTE: "Initial setpoint. The final setpoint to be determined during startup test program."

The isolation signal to RWCU-V-1 was reset and the RWCU system was returned to service.

On 4-18-84 a Leak Detection Temperature Switch monitoring the RWCU Heat Exchanger Area actuated and initiated a sequence of events the same as described for the 4-12-84 isolation.

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On 4-19-84 a Division I Leak Detection Temperature Switch monitoring the RWCU Pipe Routing Area actuated and initiated a sequence of events the same as described for the 4-12-84 isolation. The only difference was that Division I closes RWCU-V-4, the Outboard Containment Isolation Valve.

On 8-14-84 Division I and Division II Leak Detection (LD) System temperature switches actuated and initiated RWCU-V-1 and RWCU-V-4 closures. The sequence of events was the same as described for the 4-12-84 event except the setpoints (124°F) are now above those shown in Table 3.3.2-2.

On 8-31-84 a Division I LD System Temperature Switch monitoring the RWCU pipe routing area actuated. The sequence of events was the same as described in the 4-12-84 isolation. The only differences were that RWCU-V-4 closed on the Division I isolation signal and the temperature switch setpoint was adjusted to 124°F.

On 9-5-84 Division I and Division II LD System temperature switches actuated and initiated RWCU-V-1 and RWCU-V-4 closures. The sequence of events was the same as described for the 4-12-84 event except the setpoints were set to 130°F initially and have been recalibrated at 124°F (above those values shown in Table 3.3.2-2).

When the final temperature switch setpoints have been determined, a supplement to this LER will be submitted and it will include the isolations reported in this LER and any subsequent isolations resulting from Leak Detection Temperature Monitors which have not been adjusted to their final setpoint.

These events posed no actual or potential safety problem as the leak detection interlocks generated isolation signals as designed and Plant Operators took the appropriate post isolation action.

Washington Public Power Supply System
P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397
September 17, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-033-02

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-033-02 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notifications given at 1045 hours on August 31, 1984 and 1752 hours on September 5, 1984.

Very truly yours,

CM Powers for

J. D. Martin (M/D 927M)
WNP-2 Plant Manager

RLK:mm

Enclosure:
Licensee Event Report No. 84-033-02

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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