NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Dresden Nuclear Power Station	0 5 0 0 0 2 4 9	1 OF 0 2

TITLE (4)

Reactor Scram on Low Water Level

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									N/A	0 5 0 0 0
0 8	2 1	8 4	8 4	— 0 1 0	— 0 0	0 9	1 0	8 4	N/A	0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 9 7	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Lawrence Coyle (X-483)	8 1 5 9 4 2 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	
X	S J	I F C V	C 6 6 5	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During normal operation, the "A" feedwater regulating valve operator vibrated loose from the valve stem coupling. The valve disc failed in the closed direction, causing the reactor to scram on low water level. Safety significance was minimal, since all emergency systems operated as designed. Previous similar occurrence reported by R.O. 84-09 on Docket 50-237.

The operator and valve stem were reconnected and holes were drilled into the coupling block and locknuts so set screws could be placed to secure the valve stem and valve operator to the coupling block.

IE22
1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Dresden Nuclear Power Station Unit 3	0 5 0 0 0 2 4 9	8 4	— 0 1 0	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal operation the "A" feedwater regulating valve (A03-642A) operator separated from the valve stem coupling. The valve disc failed in the closed direction, causing a reactor scram on low water level. Safety significance was minimal, since all emergency systems operated as designed. Previous similar occurrence reported by R.O. 84-09 on Docket 50-237.

The cause of the event was the loosening, due to vibration, of the locknut securing the operator to the coupling block at the top of the valve stem. The loose locknut permitted the operator to vibrate loose from the coupling block allowing the coupling block and the attached valve stem to fall into the closed position. The valve operator was reattached to the coupling block. To prevent a recurrence holes were drilled into the top and bottom side of the coupling block and set screws were installed to wedge against the valve operator and the valve stem. Also, holes were drilled into the top and bottom locknuts and set screws were installed to wedge against the top and bottom of the coupling block to prevent the locknuts from moving. This procedure was repeated on the "B" feedwater regulating valve. Set screws will be installed in the Unit 2 "A" feedwater regulating valve coupling block and associated locknuts during the fall 1984 refuel outage. The 2"B" feedwater regulating valve is of a different type, and will not require set screws.

Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

September 13, 1984

DJS Ltr #84-908

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #84-010-0, Docket #050-249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(iv).

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

IE22
1/1