

Docket No. 50-346

License No. NPF-3

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RICHARD P. CROUSE
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Director of Nuclear Reactor Regulation
Attention: Mr. John F. Stolz
Operating Reactor Branch No. 4
Division of Operating Reactors
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Stolz:

This is in response to TMI Action Item II.K.3.5 "Automatic Trip of Reactor Coolant Pumps" Generic Letter No. 83-10 (G.L. 83-10). Toledo Edison with other members of the B&W Owners Group have participated and endorse the report "Analytical Justification for the Treatment of Reactor Coolant Pumps During Accident Conditions" (B&W Document No. 77-1149091-00). This generic report addresses the treatment of reactor coolant pumps during a small break loss of coolant accident (SBLOCA) for 177 fuel assembly raised and lower loop units. The treatment for the steam generator tube rupture events in the above report is only applicable to lower loop plants.

Attached is a copy of "Best Estimate Steam Generator Single Double-Ended Tube Rupture Analysis" (B&W Document No. 77-1152840-00) for the raised loop plant. This report together with the generic report provide Toledo Edison response for the treatment of reactor coolant pumps during transients and accidents. The analysis supports manually tripping of reactor coolant pumps on indication of "loss of subcooling margin". This will be incorporated into our Abnormal Transient Operating Guidelines Program (ATOG). The operator training on reactor coolant pump ATOG procedures will be completed by the end of the 1984 refueling outage.

Very truly yours,

RPC:GAB

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cc: DB-1 Resident Inspector

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