Commonwealth Edison
Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

GCT-92-016

April 2, 1992

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of March 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

Gerald C. Tietz
Technical Superintendent

GCT/CALS/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MARCH 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

On March 1, Unit One dropped to 450 MWe for "B" Gland Steam Exhaust Repairs. Load drops occured on March 8 to 650 MWe and March 15 to 328 MWe. These drops were for CIV testing and Feedwater Heater maintenance respectively. A final load drop occured or March 22 to 450 MWe for 1B Gland Steam Condenser Level Control Valve repairs.

B. Unit Two

Unit Two continued scheduled refuel outage.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Admendment No. 130 was issued on February 21, 1992 to Facility Operating License No. DPR-30 for Quad Cities Nuclear Power Station, Unit 2. This amendment revises the Technical Specifications to reflect the addition of 2 containment isolation valves, and a change in logic, to the HPCI System.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q98670	6600	Repair soak back oil pump contactor on 1/2 Diesel Generator.	As Found: Small piece of insulation and tape stuck by contractor hinge caused the contactor to chatter. As Left: Removed foreign material from contactor then energized it. Contactor operated normally.
Q90500	5600	Repair cracked insulation near lugs on heater coil 1HC on the 1/2 Diesel Generator control panel.	As Found: Found brittle insulation around conductor. As Left: Removed old wiring from TRF10-X2, 1HC-HC and HD, and TB25C-11. Replaced the internal wiring with S15 wire and relabled the wiring.
Q98377	6700	Replace secondary disconnects on 4 KV breaker #35.	As Found: Found broken phenolic on SBM switch and magnetic field side secondary disconnect. There was scarfed insulation on several wire runs. As Left: Removed magnetic field side secondary disconnect and installed new disconnect. Replaced wire from MF2-SBMB and MR4-SBMG. Removed old SBM switch and installed new SBM switch.
Q98890	7300	Repair breaker in Bus 29 cross tie has a defective secondary disconnect.	As Found: Found terminal A1 sticking in contact block. As Left: Disassembled secondary contact block and remove defective A1 contact assembly. Replace assembly with a new contact assembly.
Q97082	7500	Repair leak on the glove port opening for rough prefilter.	As Found: Found cover gasket badly worn and not enough adjustment left in clamps. As Left: Caulked around opening before reinstalling cover and replaced adjustment screws in clamps.
Q84416	0020	Install aux. steel grating support brackets per ECN 04-00087M.	As Found: Found grating in bad condition. As Left: Angles welded to beam and grating supported by same.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q83650	0030	Repair Cable jackets found damaged during cleaning removal of EHC fluid.	EHC damaged cable jackets repaired using 3 half lapped layers of scotch 130c and 1 half lapped layer of scotch 33t.
Q82801	0030	Inspect and repair as necessary, cables damaged by EHC fluid at Cable Tray 94713.	Cleaned tray, cleaned and inspected all cables. Temporarily ID'ed damaged cables for future repair.
Q82802	0030	Inspect and repair as necessary cable damaged by EHC Fluid at cable tray 947M5.	Cleaned cable tray Node 947M5 and repaired 3 damaged cables.
Q82803	0030	Inspect and repair as necessary cable damaged by EHC fluid at cable tray 947M2.	Cleaned cables and tray, also repaired 2 damaged cables.
Q82804	0030	Inspected and repair as necessary cables damaged by EHC fluid at cable tray 947M1.	Cleaned cables and tray in 947M1. Temporarily labeled damaged cables for future repair.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q97739	0260	Repair APRM 4-6 Flow Converter.	As Found: Found converter damaged and zero adjustment not working. While converter was being tested the circuit board began to smoke. As Left: Replaced converter with one from Dresden which was calibrated in IM shop. Rejected old converter to vendor for refurbishment under Work Request Q98716.
Q98710	0261	Repair U2 MSIV Temperature switch conduit fitting.	Replaced fitting with 3/4 in. conduit Pulling 90.
Q98149	0261	Repair RHR Recirc Pump differential pressure switch.	As Found: Found torque tube on bellows assembly was binding no repeatability on instrument. As Left: Replaced switch assembly and bellows assembly. Also calibrated switches and indicator.
Q98233	0261	Replace RHR Recirc pump differential pressure loop select logic 2-261.	As Found: Switch lock screw was broken and unable to secure switch. As Left: Removed old switch and installed replacement, installed new test fittings on low and high side, installed new caps and new tubing adapters. Calibrated switch and indication and restored to original condition.
Q97602	0263	Investigate and repair supply voltage found to be out of range, on Reactor vessel level upper 400.	Voltage is within acceptable tolerance when the transmitter is unloaded. OAD to remove 200 OHM resistor from input card under Work Request Q98903 to adjust card for signal in a 10-50 MA loop.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q98949	0300	Repair leak on accumulator for valve 2-0305-111.	As Found: Found O-Rings to be flat and pliable, some minor dirt. As Left: Wiped clean and installed new O-Rings and valve. Lubricated and hand tighten torquing to 50 ft. lbs.
Q97387	0750	Investigate and repair LPRM 24-49C which has hard hi signal.	As Found: LPRM is bypassed, and no longer reads upscale when not in bypass. As Left: Did TDR traces, appears problem could be under pot or detector. Replaced connector under pot using Work Request Q92438. LPRM is now reading zero.
Q99033	0902	Trouble shoot push button on 902-5 panel.	As Found: Found relay was loose in its socket. As Left: Replaced relay and tested good.
Q98251	1000	Repair limitorque for RHR torus dump valve, unit will not declutch.	As Found: Found motor pinion gear had walked of its shaft, oil seal on motor shaft leaking, and bearings were bad. As Left: Put limitorque in manual three times worked fine. Rejected motor and new motor to be installed on Q92529.
Q98702	1000	Repair contacts 12 & 12C on limitorque valve for 2B RHR suction valve.	As Found: Found terminal #12 and #13 to be open and wires 12 and 12C were crimped on insulation. As Left: Crimped new lugs on 12 and 12C cable is #14 AWG. Terminals #12 and #13 now read 1.0 OHMS. Control successfully stroked the valve.
Q98790	2400	Investigate and repair Channel A monitor reading 3 times higher than source or Channel B.	At X-20 used bias pot to adjust reading to 10R/hr (per checked exposure rate). At X-2 used slope pot to adjust reading down to 100 R/hr. (per checked exposure rate). Went back to the minimum and took "As Lefts".
Q98536	3900	Investigate and repair valve handwheel which broke off RHR Service Water low pressure piping.	As Found: Found valve handwheel and top of stem broken off. As Left: Replaced valve bonnet assembly with new one.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q98535	5200	Repair fuel oil leak.	As Found: Nipple leaking on fuel pump diesel line. Down stream brass 1/2 inch pipe nipple was cracked. As Left: Replaced both nipples where leak was detected. Found leak at brazed "T" nipple. New nipple was fabricated and installed by ENC under Work Request Q98560.
Q90858	5700	Disassemble 2A RHR Room Cooler for inspection by system engineer.	As found: Cooler plugged with mud. As Left: Cleaned cooler and reassembled it.
Q99216	6600	Remove thermocouple which is broken of in #8 cylinder for U2 Diesel Generator.	As Found: Thermocouple had been removed from #8 cylinder exhaust chamber but thermocouple fitting was seized in place. As Left: Strong backs were fabricated and placed over the remaining piece of the thermocouple fitting to seal off the exhaust.
Q98252	6600	Investigate why U2 Diesel Generator voltage regulator KVARS swung excessively with Diesel Generator loaded.	As Found: Visual inspection of voltage regulator found no problems. As Left: After visual inspection U2 Diesel Generator was run loaded and performed properly on 2-19-92 and 2-29-92.
Q98736	6700	Relug and reland white-black conductor to terminal ZX-2, Cable 20661 on Cubicle 8 Bus 23.	As Found: Found wire was in lug but pulled right out. As Left: Relugged the white-black conductor to terminal 2W-2 in Cubicle 8, Bus 23. Removed to ty-raps and replaced with new.
Q97890	6700	Repair breaker on Bus 23-1 Cubicle 3.	Adjusted jack nut in Cubicle 3 for proper breaker operation and cleaned up cable side covers for cables so breaker can be installed properly.
Q98243	0830	Replace breaker for MCC 2A compt. on RWCU Recirc shutoff.	As Found: Old breaker failed trip check. As Left: Installed new breaker and tested good.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

Licensee Event Report Number	Date	Title of Occurrence
92-007	02-28-92	CR Vent Isolation from CL detector
92-008	03-11-92	1A & 1B RHR Room Coolers plugged. This happened on 11/24/90 and 1/5/91.

UNIT 2

92-007	03-04-92	2B RHR Hx Cooler tubes plugged.
92-008	03-16-92	ESF Actuation while conducting RCIC logic test (ECCS Initiation)
92-009	03-26-92	Auto start of 2C and 2D RHR pumps.
92-010	03-29-92	Low temperature on RPV bottom head during hydro test.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE April 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 2400 030192 030192 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	1911.3	138422.4
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3421.9
7. HOURS GENERATOR ON LINE	744.0	1892.0	134123.1
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)	1789300.0	4598294.4	288648324.4
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	587483.0	1505068.0	93579404.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	565454.0	1461730.0	88176398.0
12. REACTOR SERVICE FACTOR	100.00	87.51	79.07
13. REACTOR AVAILABILITY FACTOR	100.00	87.51	81.03
14. UNIT SERVICE FACTOR	100.00	86.63	76.62
15. UNIT AVAILABILITY FACTOR	100.00	86.63	77.14
16. UNIT CAPACITY FACTOR (Using MDC)	98.63	87.35	65.50
17. UNIT CAPACITY FACTOR (Using Design MWe)	96.33	85.14	63.84
18. UNIT FORCED OUTAGE RATE	0.0	13.37	5.84

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE April 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 2400 030192 033192 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.25	133482.95
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	2985.8
7. HOURS GENERATOR ON LINE	0.0	0.25	130020.15
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	702.9
9. GROSS THERMAL ENERGY GENERATED (MWH)	0.0	175.2	280084062.20
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.0	50.0	89930240.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	-5977.0	-15809.0	85118366.0
12. REACTOR SERVICE FACTOR	0.	0.01	76.94
13. REACTOR AVAILABILITY FACTOR	0.0	0.01	78.66
14. UNIT SERVICE FACTOR	0.0	0.01	74.94
15. UNIT AVAILABILITY FACTOR	0.0	0.01	75.35
16. UNIT CAPACITY FACTOR (Using MDC)	-1.04	-0.94	63.80
17. UNIT CAPACITY FACTOR (Using Design MWe)	-1.02	-0.92	62.18
18. UNIT FORCED OUTAGE RATE	0.0	0.0	8.09

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: April 10, 1992
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE April 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH March 1992

DAY	AVEREAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	560	17.	793
2.	791	18.	794
3.	793	19.	793
4.	792	20.	793
5.	793	21.	784
6.	794	22.	638
7.	794	23.	791
8.	759	24.	791
9.	793	25.	792
10.	791	26.	791
11.	791	27.	790
12.	793	28.	786
13.	795	29.	788
14.	794	30.	786
15.	334	31.	787
16.	685		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE April 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH March 1992

DAY	AVEREAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-8	17.	-8
2.	-8	18.	-8
3.	-8	19.	-8
4.	-8	20.	-8
5.	-8	21.	-8
6.	-8	22.	-8
7.	-8	23.	-8
8.	-8	24.	-8
9.	-8	25.	-8
10.	-8	26.	-8
11.	-8	27.	-8
12.	-8	28.	-8
13.	-8	29.	-8
14.	-8	30.	-8
15.	-8	31.	-8
16.	-8		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME Unit One

DATE April 2, 1992

REPORT MONTH March 1992

COMPLETED BY Matt Benson

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-03	3-15-92	S	22	B	5	- - - -	- -	- - - -	Maintenance performed on Unit 1 Feedwater Heater

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Unit 2

COMPLETED BY Matt Benson

DATE April 2, 1992

REPORT MONTH March 1992

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-01	3-1-92	S	744	C	4	- - - -	- - -	- - - -	Continuation of Unit 2 Scheduled Refuel Outage.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive scram timing data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 11 Cycle: 12

2. Scheduled date for next refueling shutdown: 9-5-92

3. Scheduled date for restart following refueling: 12-5-92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   NOT AS YET DETERMINED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1405

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licens storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

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O.C.O.S.R.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: 10 Cycle: 11

2. Scheduled date for next refueling shutdown: 01/01/92

3. Scheduled date for restart following refueling: 04/12/92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment: Yes, as listed below:
   1. Remove Table 3.7-2
   2. Modification to turbine control valve fast acting solenoid valve.
   3. Modification to HPCI turbine exhaust steam line.
   4. HPCI/RCIC 24-hour shutdown action provision.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:
   1. 01/15/92
   2. 04/18/91
   3. 06/28/91
   4. 12/31/91

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2439

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

APPROVED
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O.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - Inservice Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRs
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Fraction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center