

NUCLEAR ENGINEERING SCIENCES DEPARTMENT
Nuclear Reactor Facility
University of Florida



W.G. Varnelson, Director
NUCLEAR REACTOR BUILDING
Gainesville, Florida 32611
Phone (904) 392-1429 - Telex 56330

April 3, 1992

UFTR Safety Analysis Report
Revision 7, 4/92

U.S. Nuclear Regulatory Commission
Washington, D.C. 20555
Attn: Document Control Desk

Re: University of Florida Training Reactor
Facility License: R-56, Docket No. 50-83

Gentlemen:

The enclosed package contains Revision 7 pages for the UFTR Safety Analysis Report dated January, 1981 submitted as part of our relicensing effort. Revision 7 consists of changes to two pages. The revision has resulted from the need to make certain minor changes in the descriptions of the resins used in the primary coolant system demineralizer and makeup water system demineralizer as well as the need to make a change in the secondary cooling system pump. All changes have been reviewed by UFTR management and the UFTR Safety Review Subcommittee and are not considered to involve any unreviewed safety question or to impact the UFTR Safety Analysis as outlined below; all text changes are denoted by vertical lines in the right hand margin of the attached affected replacement pages. Reasons for all text changes are explained in the following paragraphs.

The first change is included on Page 5-8 to allow the use of an equivalent deep well pump per the slightly changed description in Section 5.2 describing the UFTR Secondary Cooling System. This change to a more efficient pump was necessitated by the failure of the previous pump in February, 1992 and the unavailability of an exact replacement; it was evaluated and determined not to involve any unreviewed safety questions per 10 CFR 50.59 Number 92-01.

The second change is included on Page 9-6 because the Amberlite IRN-150, nuclear-grade resins specified for use in the Demineralized Water Makeup System and the primary coolant Purification System are no longer available. These systems were converted to utilizing equivalent Purolite NRW-37 resins in January, 1991; however, the changes in Section 9.2.3 and Section 9.2.4 are included to allow the use of any equivalent resin. In this way any future substitution can be made following an in-house evaluation of equivalency. This change was evaluated and determined not to involve any unreviewed safety question per 10 CFR 50.59 Number 91-01.

9204150014 920403
PDR ADDCK 05000083
P PDR

Equal Opportunity/Affirmative Action Employer

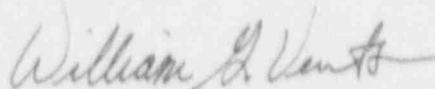
Put No
P852203144
A020
11

U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
April 3, 1992
Page 2

As indicated, these Revision 7 changes have been fully reviewed by UFTR Management and the Reactor Safety Review Subcommittee to involve no unreviewed safety question per 10 CFR 50.59 evaluations and determinations and so are not considered to relax the requirements for assuring protection of the health and safety of the public and of the reactor facility. The changes simply update the SAR.

The entire enclosure consists of one(1) signed original letter of transmittal with enclosure plus ten(10) copies of the entire package. If further information is required, please advise.

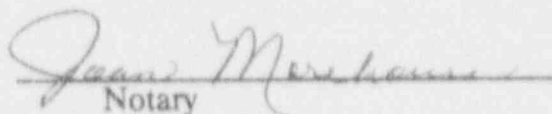
Sincerely,



William G. Vernetson
Associate Engineer and
Director of Nuclear Facilities

WGV/p
Enclosures

cc: U.S. NRC Region II
R. Piciullo
Reactor Safety Review Subcommittee


Notary

4/3/92
Date