

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Beaver Valley Power Station, Unit 1DOCKET NUMBER (2)  
0 5 0 0 0 3 3 4 1 OF 0 2TITLE (4)  
Failure of Containment (CNMT) Recirculation Cooling Coils Chilled Water  
System Outlet Isolation Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)							
0	7	0	4	8	4	8	4	0	0	7	0	0	0	0	0	0		
N/A												0	5	0	0	0	0	0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											
POWER LEVEL (10)	1	0	0	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)						
				20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)						
				20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
				20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
				20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)											

LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Robert J. Druga, Chief Engineer  
TELEPHONE NUMBER  
AREA CODE  
4 1 2 6 4 3 1 - 1 2 6 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM
X	K	M	I	S	V	X	M	1	2	0	N

SUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 7/4/84, at 2138 hours, during normal operations, the Containment Recirculation Cooling Coils Chilled Water System Outlet Isolation Valve failed shut. This caused a loss of cooling water flow to the CNMT Air Recirculation Cooling Coils and to the CNMT Instrument Air Compressors. The loss of cooling water to the Air Recirculation Cooling Coils resulted in increasing CNMT temperatures. Subsequent attempts to restore cooling water were unsuccessful and at 2238 hours, CNMT temperature reached 105.08 degrees. Technical Specification 3.6.1.5 requires the CNMT temperature to be less than 105 degrees. Station Management then elected to reduce power to effect valve repairs and to reduce CNMT temperature. At 2315 hours, the Instrument Air to CNMT Instrument Air Isolation Valve was opened to supply CNMT with Instrument Air. This was done due to the loss of cooling to the CNMT Instrument Air Compressors. At 2318 hours, CNMT temperature reached 106.38 degrees. A manual shutdown to Hot Standby was commenced. The cause for the Chilled Water System Isolation Valve failing shut was due to a failed pneumatic valve diaphragm. This diaphragm was replaced. There were no safety implications to the public because the reactor was placed in a safe, controlled shutdown condition and the River Water System was operable at all times as an additional source of cooling if necessary.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Beaver Valley Power Station, Unit 1	0 5 0 0 0 3 3 4	8 4	— 0 0 7	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 7/4/84, at 2138 hours, during normal operations at 100% reactor power, the Containment Recirculation Cooling Coils Chilled Water System Outlet Isolation Valve [TV-CC-110D] failed shut. This caused a loss of cooling water flow to the Containment Air Recirculation Cooling Coils and the Containment Air Compressors. The loss of cooling water to the Containment Air Recirculation Cooling Coils resulted in increasing Containment temperatures. Subsequent attempts to open the isolation valve [TV-CC-110D] were unsuccessful and at 2238 hours, Containment temperature reached 105.08 degrees. Technical Specification 3.6.1.5 requires the Containment temperature to be less than 105 degrees. Station management then elected to reduce power to effect valve repairs and to attempt to reduce Containment heat load. At 2311 hours, a Containment entry was made to investigate the failure of [TV-CC-110D]. At 2315 hours, the Instrument Air to Containment Instrument Air Isolation Valve [IA-90] was opened to supply Containment with Instrument Air. This was done due to the loss of cooling water to the Containment Air Compressors. At 2318 hours, Containment temperature reached 106.38 degrees. A controlled manual shutdown to Hot Standby was commenced due to the increasing Containment temperatures. On 7/5/84, two additional Containment entries were made to investigate the failure of [TV-CC-110D]. It was determined that [TV-CC-110D] failed shut due to a failed pneumatic valve diaphragm. An additional Containment entry was made on 7/5/84 to erect scaffolding to effect repairs on [TV-CC-110D]. On 7/7/84 at 2000 hours, the valve operating diaphragm and air regulator on [TV-CC-110D] were replaced. No other corrective actions are planned.

There were no safety implications to the public because the reactor was placed in a safe shutdown condition and the River Water System was operable at all times as an additional source of cooling if necessary.

The Containment Recirculation Cooling Coils Chilled Water System Outlet Isolation Valve [TV-CC-110D] is a Masoneillan Trip Valve, Model No. 38-20761. This is the first reported failure of this valve.

RECEIVED-REGION 1

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**Duquesne Light**

Nuclear Division  
P.O. Box 4  
Shippingport, PA 15077-0004

Telephone (412) 393-6000

August 1, 1984  
ND1SS1:2130

Beaver Valley Power Station, Unit No. 1  
Docket No. 50-334, License No. DPR-66  
LER 84-007

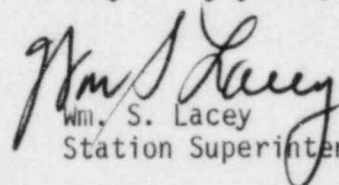
Dr. Thomas E. Murley  
Regional Administrator  
United States Nuclear Regulatory Commission  
Region 1  
Park Avenue  
King of Prussia, PA 19406

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 84-007, 10 CFR 50.73.a.2.i, "Completion of Nuclear Plant Shutdown required by Technical Specifications."

Very truly yours,

  
Wm. S. Lacey  
Station Superintendent

Attachment

11  
1E 22

T. E. Murley  
July 20, 1984  
ND1SS1:2130  
Page two

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Washington, D.C. 20555

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