

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Beaver Valley Power Station, Unit 1DOCKET NUMBER (2)  
0 5 0 0 0 3 3 4PAGE (3)  
1 OF 01TITLE (4)  
Inadvertent Reactor TripEVENT DATE (5)  
MONTH DAY YEAR  
0 7 0 6 8 4 8 4  
LER NUMBER (6)  
YEAR SEQUENTIAL NUMBER REVISION NUMBER  
0 0 0 8 0 0 0 8 0 1 8 4  
REPORT DATE (7)  
MONTH DAY YEAR  
0 7 0 6 8 4 8 4  
OTHER FACILITIES INVOLVED (8)  
FACILITY NAMES  
DOCKET NUMBER(S)  
N/A 0 5 0 0 0  
N/A 0 5 0 0 0THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)  
OPERATING MODE (9) 5  
POWER LEVEL (10) 0 0 10  
20.402(b) 20.406(c) X 60.73(a)(2)(iv) 73.71(b)  
20.406(a)(1)(i) 60.36(a)(1) 60.73(a)(2)(v) 73.71(c)  
20.406(a)(1)(ii) 60.36(a)(2) 60.73(a)(2)(vi) OTHER (Specify in Abstract below and in Text, NRC Form 365A)  
20.406(a)(1)(iii) 60.73(a)(2)(i) 60.73(a)(2)(vii)(A) 60.73(a)(2)(viii)(B)  
20.406(a)(1)(iv) 60.73(a)(2)(ii) 60.73(a)(2)(viii)(B)  
20.406(a)(1)(v) 60.73(a)(2)(iii) 60.73(a)(2)(x)LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Robert J. Druga, Chief Engineer  
TELEPHONE NUMBER  
AREA CODE  
4 1 2 6 4 3 - 1 2 6 4COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)  
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRCDS  
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRCDSSUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2140 hours, during the performance of Maintenance Surveillance Procedure 26.01 (P-446, Turbine First Stage Pressure Protection Channel III Test), the reactor shutdown bank rods tripped on a pressurizer low pressure trip signal. The trip occurred when the technicians input a test signal for the turbine first stage pressure channel. When this was done, the P-7 interlock (which unblocks the pressurizer low pressure reactor trip above 10 percent power) was activated and caused the reactor trip. Because the plant was in cold shutdown, no action beyond the immediate verification of the trip was necessary.

It was later discovered that, contrary to the initial conditions of MSP 26.01, the test was performed with the reactor trip breakers closed. The Shift Supervisor and I&C personnel have been counseled as to the importance of thoroughly reviewing all procedures prior to their performance. An evaluation of the procedure in question has been made and it was judged that in this case, a procedure change is necessary for clarification.

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PDR ADOCK 05000334  
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**Duquesne Light**

Nuclear Division  
P.O. Box 4  
Shippingport, PA 15077-0004

Telephone (412) 393-6000

August 1, 1984  
ND1SS1:2131

Beaver Valley Power Station, Unit No. 1  
Docket No. 50-334, License No. DPR-66  
LER 84-008

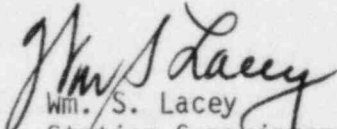
Dr. Thomas E. Murley  
Regional Administrator  
United States Nuclear Regulatory Commission  
Region 1  
Park Avenue  
King of Prussia, PA 19406

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 84-008, 10 CFR 50.73.a.2.iv, "Engineered Safeguards Features System Actuation."

Very truly yours,

  
Wm. S. Lacey  
Station Superintendent

Attachment

T. E. Murley  
July 20, 1984  
ND1SS1:2131  
Page two

cc: Director of Management & Program Analysis  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

C. A. Roteck, Ohio Edison

Director, Office of Inspection and Enforcement Headquarters  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

Mr. Peter Tam, BVPS Licensing Project Manager  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

W. Troskoski, Nuclear Regulatory Commission, BVPS Site Inspector

Mr. Alex Timme, CAPCO Nuclear Projects Coordinator, Toledo Edison

INPO Records Center  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, GA 30339

G. E. Muckle, Factory Mutual Engineering, Pittsburgh

Mr. J. A. Triggiani, Operating Plant Projects Manager  
Mid Atlantic Area  
Westinghouse Electric Corporation  
Nuclear Services Integration Division  
Box 2728  
Pittsburgh, PA 15230

American Nuclear Insurers  
c/o Dottie Sherman, ANI Library  
The Exchange Suite 245  
270 Farmington Avenue  
Farmington, CN 06032