NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	1 OF 0 2

TITLE (4)
Unscheduled Actuations of Control Room Emergency Filtration Unit

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
											0 5 0 0 0
0 7	2 0	8 4	8 4	-	0 7 7	- 0 0	0 8	0 8	8 4		0 5 0 0 0

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
4	0 0 0	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)	X	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		50.72(b)(2)(ii)
		20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
R. L. Koenigs, Compliance Engineer	5 0 9 3 7 7 - 1 2 5 0 1
	Ext. 2279

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	IL	R A	K 0 2 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A Control Room Emergency Filtration Unit (an ESF system) was automatically actuated on 7/20/84 and 7/21/84 due to spikes on a corresponding Control Room Outside Air Radiation Monitor.

In response to each event, after verifying that radiation levels were not above normal background, the emergency filtration unit and the radiation monitor were reset and returned to a normal lineup.

These events were verbally reported to the NRC (2339 hours 7/20/84 and 1250 hours 7/21/84) in accordance with 10CFR50.72(b)(2)(ii).

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Conditions

a) Plant Mode - 4
b) Power Level - 0

Event

The Division II Control Room Emergency Filtration Unit (WMA-FN-54B) was automatically actuated on 7/20/84 (2323 hours) and on 7/21/84 (1145 hours) by High-High radiation alarms originating from an outside air intake radiation monitor (WOA-RIS-31B).

Immediate Corrective Action

Normal background levels were observed at the Division I and Division II radiation monitors. The associated recorder (WOA-RR-32) in each case revealed that monitor 31B and received a spurious spike of sufficient magnitude to trip the High-High alarm. The alarms were promptly reset and the EST system returned to normal. Notification was given to the NRC in accordance with the requirements of 10CFR50.72(b)(2)(ii).

Long Term Corrective Action

The spike on 7/20/84 was found to have been caused by the remote operation of a reactor core isolation cooling valve (RCIC-V-113). The operation of this valve had previously caused ESF system actuation (Ref. LER 84-030). Installation design of previously installed noise suppression devices was determined to have been incorrect and are currently being reworked.

Investigations failed to determine the origin of the spike on 7/21/84. Investigation and resolution of noise problems are continuing on the Radiation Monitoring and interfacing systems (Ref. LER 84-067).

Safety Significance

There were no safety consequences associated with these events. All Plant systems performed as required during the events.

Washington Public Power Supply System
P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

August 8, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-077

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-077 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1250 hours on July 21, 1984.

Very truly yours,

J. D. Martin

J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 84-077

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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