NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Turkey Point Unit 3	0 5 0 0 0 2 5 0	1 OF 0 2

TITLE (4)
Technical Specification - Reactor Coolant System Leakage

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 7	1 2	8 4	8 4	-	0 2 0	- 0 0	0 8	1 0	8 4	Turkey Point Unit 4	0 5 0 0 0 2 5 1
										N/A	0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	1 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
John Lovell, Regulation and Compliance Engineer	AREA CODE 3 0 5 2 4 5 - 2 9 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	A B	R T V	R 3 4 4	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO		0 3	1 5	8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 12, 1984, Unit 3 was shutdown from 100% power due to a Reactor Coolant System (RCS) leak of approximately 13.5 gpm. The cause was leakage due to a broken gland flange on valve 3-538, the lower root valve on the pressurizer level instrument sensing line to LT-3-459. The affected loop bistables were tripped in accordance with Operating Procedure 0208.14. Therefore, the Technical Specification requirement for minimum degree of redundancy for reactor trip signals on pressurizer high water level was satisified. During a RCS cooldown to affect repairs, valve 3-538 was manually backseated and the leak stopped. Immediate corrective actions included: 1) a manual unit shutdown and subsequent cooldown to repair valve, 2) original valve packing gland flange was replaced with a "strong-back" plate and washer, 3) an inspection of all Rockwell 3/4 inch valves on Units 3 and 4 with both units shutdown and, 4) an overpressure test and visual leak check of the RCS were performed and satisfactorily completed. The long term corrective action to be taken is to have engineering evaluate these failures for the root cause and provide permanent fix recommendations. The health and safety of the public were not affected. Similar occurrences: 250-84-019

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Turkey Point Unit 3	0 5 0 0 0 2 5 0	8 4	0 2 0	0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On July 12, 1984, at 5:20 p.m., a reactor shutdown was commenced on Unit 3 due to a Reactor Coolant System (RCS) leak of approximately 13.5 gpm. The cause was a packing leak on the lower isolation valve, 3-538, on the instrument sensing line to pressurizer level transmitter LT-3-460. The affected loop bistables were tripped in accordance with Operating Procedure 0208.14. Thus satisfying the Technical Specification requirement for minimum degree of redundancy for reactor trip signals on pressurizer high water level.

The first indications of a leak were noted approximately 20 minutes after the pressurizer level transmitter LT-3-459 failed low. The sump level showed an increased rate of rise and, containment pressure showed a slight upward trend. The RCS leak rate, using the volume control tank (VCT) and containment sump levels, was estimated at approximately 13.5 gpm. A manual unit shutdown was commenced and the unit removed from service at 7:52 p.m. per Technical Specification 3.1.3. The leak was identified as a broken gland flange on valve 3-538 during a containment entry. A RCS cooldown was initiated to affect repairs on the valve. A second containment entry resulted in isolating the leak by fully backseating valve 3-538 at approximately 4:33 a.m., July 13. The cooldown was terminated and preparations made to repair valve 3-538 at hot shutdown conditions. A plant change modification (PCM 84-129) was prepared, reviewed, and approved to fabricate and install a "strong-back" plate and washer to replace the original valve packing gland flange that was damaged. The replacement parts were fabricated, the valve repacked, the "strong-back" and washer installed, bolted in place, and torqued to comply with Engineering's recommendations. Subsequently, an inspection of all Rockwell Edwards 3/4 inch globe valves was conducted on both Units 3 and 4. The inspection resulted in five additional gland flanges being found as unacceptable. Three valve gland flanges were cracked and two were degraded. Incorporation of the strong-back device, was affected on these valves as well. An evaluation is underway to determine the root cause of these similar failures and a satisfactory permanent fix. Level transmitter 3-459 was returned to service at 10:45 a.m., July 13. An overpressure test and visual leak inspection was successfully performed per Operating Procedure 1004.1. The unit was returned to service at 12:05 p.m., July 17, and full power operation achieved at 4:26 p.m., of the same day.

FPL

FLORIDA POWER & LIGHT COMPANY

August 10, 1984
PNS-LI-84-283

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Re: Reportable Event 84-20
Turkey Point Unit 3 50-250
Date of Event: July 12, 1984
Technical Specification -
Reactor Coolant System Leakage

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR to provide notification of the subject event.

Very truly yours,

for J. W. Williams, Jr.
Group Vice President
Nuclear Energy

JWW/PLP/js

Attachment

cc: J. P. O'Reilly, Region II, USNRC
Harold F. Reis, Esquire
File 933.1

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