



Entergy Operations, Inc.
River Bend Station
5485 U.S. Highway 61
P.O. Box 220
St. Francisville, LA 70775
Tel 504 336 6225
Fax 504 635 5068

James J. Fisicaro
Director
Nuclear Safety

November 22, 1995

U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Stop P1-37
Washington, DC 20555

Subject: River Bend Station
Docket No. 50-458
License No. NPF-47
Request for Additional Information pertaining to Licensing Amendment Request
(LAR) 95-04, Change to Technical Specifications Concerning Fuel Handling
Accident

File Nos.: G9.5, G9.42

Reference: RBG-41728, Licensing Amendment Request (LAR) 95-04, Change to Technical
Specifications Concerning Fuel Handling Accident, dated August 17, 1995

RBG-42201
RBF1-95-0284

Gentlemen:

On August 17, 1995, Entergy Operations, Inc. (EOI) submitted License Amendment Request (LAR) 95-04 (referenced letter). The LAR consisted of a proposed change to the Technical Specifications for River Bend Station (RBS) concerning engineered safety systems which respond to Fuel Handling Accident conditions.

On November 16, 1995, the NRC staff and RBS personnel discussed LAR 95-04. During the discussion, the NRC raised three questions regarding the LAR. The questions and EOI's responses are contained in the attachment to this letter. The questions and answers have been discussed with RBS's NRR Project Manager.

The information contained in this letter does not impact the scope of the original submittal; therefore, it is bounded by the No Significant Hazards Considerations previously submitted in RBG-41728.

9511280091 951122
PDR ADDCK 05000458
P PDR

ADD 1

Request for Additional Information pertaining to LAR 95-04

RBG-42201

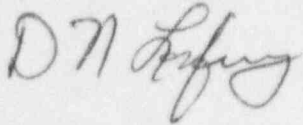
RBF1-95-0284

November 22, 1995

Page 2 of 2

If you have any questions or comments, please contact Mr. Guy H. Davant of my staff at (504) 336-6223.

Very truly yours,



Fool JJF/ghd
attachment

cc: Mr. David L. Wigginton
U. S. Nuclear Regulatory Commission
11555 Rockville Pike
M/S OWFN 13-H-3
Rockville, MD 20859

NRC Resident Inspector
P. O. Box 1051
St. Francisville, LA 70775

U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011

**RESPONSES TO NRC QUESTIONS
PERTAINING TO LAR 95-04**

1. Did RBS "do an iodine protection factor" in accordance with Murphy and Campe (M&C)?

Response:

RBS analysis with respect to the calculation of main control room (MCR) doses was performed in accordance with the intent of the M&C protection factor concept. This approach did not follow exactly the methodology as prescribed by M&C because M&C is a non-computerized procedure, whereas the approach followed by EOI is a more rigorous computerized methodology. EOI's methodology takes into account charcoal filtration of post-accident iodine nuclides both in the MCR intake pathway as well as in the recirculation pathway.

2. SAR Section 15.7, Table 11, lists the MCR volume as 240,702 cu. ft. and the ventilation flow rate as 4,000 cfm. The calculation lists the MCR volume as 188,000 cu. ft. and the flow rates -- intake, exhaust, and recirculation -- as 1,947.6 cfm. Explain the differences in the MCR volume and flow rate.

Response:

SAR Section 15.7 and associated Table 15.7-11, Fuel Handling Accident Parameters Tabulated for Postulated Accident Analysis, are based on assumptions for MCR volume and ventilation flow rates in the LOCA analysis (see Section 15.6 and Table 15.6-5). EOI's review of Question 2 indicates that when the LOCA analysis was performed during the FSAR stage, the final details of the MCR post-accident ventilation design had not been finalized. Based on available information at that time, no recirculation was contemplated. The ventilation scheme was to be a simple filtered intake/exhaust design with a ventilation flow rate of 4,000 cfm and a volume of 240,702-cu. ft. consisting of the entire MCR pressure envelope volume (comprised of the equipment room, general area corridor, and MCR itself).

Since post-LOCA MCR doses were never a problem for RBS, there was no reason to revise the LOCA analysis to reflect the as-built MCR ventilation scheme. Excess conservatism in MCR volume and ventilation flow rates has been removed from subsequent calculations. The values used in support of LAR 95-04 are more realistic and more accurately model the RBS MCR.

3. Items H and I in the RBS analysis have the same flow rates. This indicates no leakage and, as noted in Question 2, disagrees with the information in the SAR. What is the in-leakage or out-leakage assumed in the calculation?

Response:

The calculation assumes unfiltered intake for the first 66 seconds. This reflects the actuation time for the filtration system. The MCR is maintained at positive pressure; therefore, it is reasonable that the model does not include additional in-leakage terms. This is also consistent with the methodology used in the calculations supporting the licensing basis.