

**OPPD**

Omaha Public Power District  
444 South 16th Street Mall  
Omaha, Nebraska 68102-2247  
402/636-2000

April 1, 1992  
LIC-92-103R

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Mail Station P1-137  
Washington, DC 20555

- References:
1. Docket No. 50-285
  2. Letter from OPPD (W. G. Gates) to NRC (Document Control Desk) dated July 1, 1991 (LIC-91-182R)
  3. Letter from OPPD (W. G. Gates) to NRC (Document Control Desk) dated December 31, 1991 (LIC-91-337R)
  4. Letter from NRC (D. L. Wigginton) to OPPD (W. G. Gates) dated March 4, 1992

Gentlemen:

SUBJECT: Results of Prototype Power Operated Relief Valve (PORV) Block Valve Steam Blowdown Testing

In Reference 2, Omaha Public Power District (OPPD) committed to purchase a prototype PORV block valve from the original equipment manufacturer and perform steam blowdown testing by December 31, 1991. This was done to close out Item II.D.1 of NUREG-0737 for Fort Calhoun Station (FCS), demonstrating the capability of the PORV block valves to isolate steam blowdown flow through a stuck open PORV. Reference 3 submitted the preliminary test report. As committed in Reference 3, enclosed is OPPD's final report on the steam blowdown testing conducted by WYLE Laboratories. The additional information requested by the NRC in Reference 4 is provided in the report. Certain references as noted in Section 10.0 of the report are included for use by NRC technical reviewers.

The test valve configured with an actuator and a motor was sent to WYLE Laboratories for steam blowdown testing during the week of November 18, 1991. The conclusions of the enclosed final test report are:

1. The test valve and actuator accurately modeled the in-plant PORV block valves in performance characteristics.
2. The steam blowdown test conditions adequately bounded the design basis conditions of the PORV block valves installed at FCS.
3. Margin exists at actuator torque switch trip for the installed PORV block valves to perform their safety function of isolating steam blowdown flow through a "stuck open" PORV.

9204060354 920401  
PDR ADDCK 05000285  
PDR

A003 1/1

U. S. Nuclear Regulatory Commission  
LIC-92-103R  
Page 2

Therefore, the subject testing confirmed that the in-plant PORV block valves will stroke closed in 10 seconds (or less) under the maximum expected differential pressure and flow conditions. All remaining NUREG-0737, Item II.D.1 issues have been addressed by this testing.

If you should have any questions, please contact me.

Sincerely

*W. G. Gates*

W. G. Gates  
Division Manager  
Nuclear Operations

Enclosures

WGG/sel

c: LeBoeuf, Lamb, Leiby & MacRae  
D. L. Wigginton, NRC Senior Project Manager  
S. D. Bloom, NRC Project Engineer  
R. D. Martin, NRC Regional Administrator, Region IV  
R. P. Mullikin, NRC Senior Resident Inspector