

UPDATE REPORT:

CONTROL BLOCK: 

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	C	B	E	P	I	2	0	0	-	0	0	0	0	0	-	0	0	3	4							4			5
7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE						30	57	CAT 58	

CON'T

0 1 7 8

REPORT SOURCE

60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During unit power operation, the snubber shaft of hydraulic snubber 1-E11-47SS326  
0 3 | broke as a result of water hammer of the A Residual Heat Removal (RHR) System steam  
0 4 | condensing piping, which occurred when the system was started to recirculate the  
0 5 | suppression pool for sampling. The snubber is located downstream of the subject  
0 6 | piping inlet pressure control valve, 1-E11-F051A. This event did not affect the  
0 7 | health and safety of the public.

Technical Specifications 3.7.5, 6.9.1.8i

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE					
C	F	E		C		S	U	P	O	R	T	D	Z				
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.									
8	1	0	4	6	/	0	1	T	1								
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
D	F	Z	Z	0	0	0	0	0	Y	Y	A	B	2	1	0		
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS																	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 A steam pocket in the subject piping, resulting from leakage past the F051A respective  
1 1 upstream isolation valve, Ell-F052A, caused the water hammer. 47SS326 was repaired,  
1 2 tested and returned to service. To help preclude future similar events, appropriate  
1 3 modifications involving these valves have been installed on both units. Additional  
1 4 corrective actions regarding this event are presently being developed.

FACILITY STATUS										% POWER										OTHER STATUS										METHOD OF DISCOVERY										DISCOVERY DESCRIPTION									
1	5	E	28	0	7	5	29	NA										B	31	Plant Surveillance																													
ACTIVITY										CONTENT										AMOUNT OF ACTIVITY										LOCATION OF RELEASE																			
1	6	Z	33	Z	34	NA										NA																																	
PERSONNEL EXPOSURES										DESCRIPTION																																							
1	7	0	0	0	37	Z	38	NA																																									
PERSONNEL INJURIES										DESCRIPTION																																							
1	8	0	0	0	40	NA																																											
LOSS OF OR DAMAGE TO FACILITY										DESCRIPTION																																							
1	9	Z	42	8408160133 840803										NA																																			
PDR										ADOCK 05000325																																							
S										PDR																																							
PUBLICITY										ISSUED										NRC USE ONLY																													
2	0	N	44	NA																																													

NAME OF PREPARER M. J. Pastva, Jr.

PHONE: 919-457-9521

NRC USE ONLY

LER ATTACHMENT - RO #1-81-46

Facility: Unit 1

Event Date: April 14, 1981

During Unit 1 power operation, water hammer of the A Residual Heat Removal (RHR) System steam condensing piping to the A RHR heat exchanger occurred when the system was started to recirculate the suppression pool for chemistry sampling. Subsequent plant surveillance revealed that hydraulic snubber 1-E11-47SS326, which restrains the subject steam condensing piping at a point downstream of the piping inlet pressure control valve, 1-E11-F051A, was inoperable due to a broken snubber shaft.

A failure analysis of the broken snubber shaft indicates the shaft breakage occurred due to shear failure. The subject piping is normally full of RHR System water up to the line isolation valve, 1-E11-F052A, which is upstream of the F051A. Leakage past both closed valves allowed a steam pocket to form in the subject line. When the RHR System A loop was started for recirculation and sampling of the suppression pool, the steam pocket was displaced by the loop water volume and water hammer of the line occurred, thereby causing the snubber shaft to break.

Following the event, the snubber, Bergen and Patterson Part No. 3 KIP HSSA-3, was repaired, satisfactorily tested, and returned to service. Once per four hours, venting of the RHR System steam condensing loop piping on both units was initiated to ensure the piping full of water.

In accordance with plant modifications, the F052A and B line isolation valves on both units were replaced to help eliminate the leakage problem. In addition, manual isolation valves will be installed in accordance with plant modifications on the respective 1" bypass lines around the F051A, B, and F052A, B valves of the A and B RHR System steam condensing loops on both units to help eliminate a source of inleakage to the loops. On both units, A and B RHR System steam condensing loops are currently being vented once per eight hours in plant operational modes requiring their operability.



Carolina Power & Light Company 7

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429  
August 3, 1984

FILE: B09-13510C  
SERIAL: BSEP/84-1149

Mr. James P. O'Reilly, Administrator  
U. S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street N.W.  
Atlanta, GA 30323

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
SUPPLEMENT TO LICENSEE EVENT REPORT 1-81-46

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.8i of the Technical Specifications for Brunswick Steam Electric Plant, Unit 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within fourteen (14) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

RMP/mag/LETGG2

Enclosure

cc: Mr. R. C. DeYoung  
NRC Document Control Desk

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