



Northern States Power Company

414 Nicollet Mall  
Minneapolis, Minnesota 55401-1927  
Telephone (612) 330-5500

March 19, 1992

10 CFR Part 50  
Section 50.73

U S Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket Nos. 50-282 License Nos. DPR-42  
50-306 DPR-60

Unplanned Auto-Start Of An Auxiliary Feedwater Pump Due To Personnel Error

The Licensee Event Report for this occurrence is attached.

This event was reported via the Emergency Notification System in accordance with 10 CFR Part 50, Section 50.72, on February 19, 1992. Please contact us if you require additional information related to this event.

Thomas M. Parker  
Manager  
Nuclear Support Services

c: Regional Administrator - Region III, NRC  
NRR Project Manager, NRC  
Senior Resident Inspector, NRC  
Dr Raymond Thron, MDH

Attachment

150004

9203260002 920319  
PDR ADOCK 05000306  
S PDR

JE22

## LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT 2

DOCKET NUMBER (2)

0 5 0 0 0 1 3 0 0 1

PAGE (3)

1 OF 03

TITLE (4)

Unplanned Auto-Start of an Auxiliary Feedwater Pump Due to Personnel Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
02	19	92	92	001	00	03	19	92	Prairie Island Unit 1		0 5 0 0 0 2 1 5 1 2
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11)								
N			20.402(b) <input checked="" type="checkbox"/> 20.405(c) <input checked="" type="checkbox"/> 50.73(a)(2)(iv) <input type="checkbox"/> 73.71(b) <input type="checkbox"/>								
POWER LEVEL (10)			20.405(a)(1)(i) <input type="checkbox"/> 50.36(c)(1) <input type="checkbox"/> 50.73(a)(2)(v) <input type="checkbox"/> 73.71(c) <input type="checkbox"/>								
10			20.405(a)(1)(ii) <input type="checkbox"/> 50.36(c)(2) <input type="checkbox"/> 50.73(a)(2)(vi) <input type="checkbox"/> OTHER (Specify in Abstract Below and in Text, NRC Form 366A) <input type="checkbox"/>								
			20.405(a)(1)(iii) <input type="checkbox"/> 50.73(a)(2)(iii) <input type="checkbox"/> 50.73(a)(2)(vii)(A) <input type="checkbox"/>								
			20.405(a)(1)(iv) <input type="checkbox"/> 50.73(a)(2)(iv) <input type="checkbox"/> 50.73(a)(2)(viii)(B) <input type="checkbox"/>								
			20.405(a)(1)(v) <input type="checkbox"/> 50.73(a)(2)(v) <input type="checkbox"/> 50.73(a)(2)(ix) <input type="checkbox"/>								

LICENSEE CONTACT FOR THIS LER (12)

NAME

Arne A Hunstad

TELEPHONE NUMBER

AREA CODE

6 1 2 3 8 8 - 1 1 2 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE):	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On February 18, 1992, Unit 2 was in hot shutdown in preparation for refueling. The main steam isolation valves had been closed, but there was some leakage by the valves, causing cooldown of the reactor coolant system. SP2103, No. 22 Turbine-Driven Auxiliary Feedwater Pump Test, was begun at 2345 hours. This test is a full flow test of the pump. The addition of cold water to the steam generators from the performance of this test aggravated the cooldown of the reactor coolant system. A result of the cooldown is that steam generator water levels shrink. The operator performing the test in the control room was cautioned by other operators to watch steam generator levels. The operator noted that there was 65% wide range level in the steam generators, and did not realize that he was being cautioned concerning the potential auto-start of No. 21 Motor-Driven Auxiliary Feedwater Pump at 13% narrow range level. At 0051 on February 19, 1992, No. 21 Motor-Driven Auxiliary Feedwater Pump started automatically. This was a non-ESF actuation of ESF equipment.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Prairie Island Nuc Gen Plant Unit 2	0 5 0 0 0 3 0 6 9 2	0 0 1	0 0	0 2	OF 0 3	

TEXT (If more space is required, use additional NRC Form 366A's) (1-7)

EVENT DESCRIPTION

On February 18, 1992, Unit 2 was in hot shutdown in preparation for refueling. The main steam isolation valves had been closed, but there was some leakage by the valves, causing cooldown of the reactor coolant system. SP2103, No. 22 Turbine-Driven Auxiliary Feedwater Pump Test (EIS System Identifier BA), was begun at 2345 hours. This test is a full flow test of the pump. The addition of cold water to the steam generators from performance of this test aggravated the cooldown of the reactor coolant system. A result of the cooldown is that steam generator water levels shrink. The operator performing the test in the control room was cautioned by other operators to watch steam generator levels. The operator noted that there was 65% wide range level in the steam generators, and did not realize that he was being cautioned concerning the potential auto-start of No. 21 Motor-Driven Auxiliary Feedwater Pump at 13% narrow range level. At 0051 on February 19, 1992, No. 21 Motor-Driven Auxiliary Feedwater Pump started automatically. This was a non-ESF actuation of ESF equipment.

CAUSE OF THE EVENT

This event had several causes:

1. Verbal Communication - The operator performing the test was told by other operators to watch the steam generator levels. When cautioned to watch steam generator levels, he did not realize they were concerned with auto-start of the other auxiliary feedwater pump since they did not specify which level indications to watch or the reason for watching the levels.
2. Written Communication - The surveillance procedure does not follow the writers' guide for "Instruction for Content of Each Section".
3. Perceived Pressure to Complete Task - The operator performing the test perceived some pressure to complete the test since the RCS was being cooled down.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATES TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1):

DOCKET NUMBER (2):

LER NUMBER (6):

PAGE (3):

Prairie Island Nuc Gen Plant Unit 2

05000306

92-001-00003 Of 03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

CORRECTIVE ACTION

1. Personnel were reminded that verbal messages need to be clear and complete to avoid misunderstanding.
2. Auxiliary feedwater pump test procedures will be revised to bring them into compliance with the writers' guide.
3. Review the conditions for performing this test to determine if there is a more advantageous time for placement of the test in the outage schedule.

ANALYSIS OF THE EVENT

Auxiliary feedwater pumps are routinely used during startup and shutdown operations. Since No. 21 Motor-Driven Auxiliary Feedwater Pump responded as designed during this event, there was no effect on the health and safety of the public.

Since this event resulted in an unplanned automatic start of an ESF component, it is reportable pursuant to 10CFR50.73(a)(2)(iv).

FAILED COMPONENT IDENTIFICATION

None.

PREVIOUS SIMILAR EVENTS

Previous unplanned auto-starts of auxiliary feedwater pumps have been reported as Unit 1 LER's 87-006 and 89-005, and Unit 2 LER's 86-004, 90-004, and 90-005.