

Northeast  
Utilities System

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November 9, 1995

Docket No. 50-336  
B15438

Re: 10CFR50.90

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 2  
Additional Information Related to  
Proposed Revision to Technical Specifications  
Pressurizer and Main Steam Safety Valves

In a letter dated September 29, 1995,<sup>(1)</sup> Northeast Nuclear Energy Company (NNECO) proposed to amend the Millstone Nuclear Power Station, Unit No. 2 Technical Specifications to expand the as-found lift pressure setpoint tolerance for the pressurizer and the main steam safety valves from  $\pm 1$  percent to  $\pm 3$  percent. In a telephone conversation on November 1, 1995, between NNECO and the NRC Staff, the Staff questioned why the analysis submitted in support of the proposed revision was limited to the Steam Generator Tube Rupture Event and why the Feedwater Line Break Event was not analyzed. The Staff noted that in similar proposals by licensees with plants designed by Combustion Engineering, the Feedwater Line Break Event was frequently determined to be the limiting pressurization event for the reactor coolant system. The purpose of this letter is to document the responses that NNECO provided in the telephone conversation with the Staff on November 1, 1995.

NNECO informed the Staff that, with the exception of the Steam Generator Tube Rupture Event, the events analyzed in the Safety Analyses section (Chapter 14) of the Millstone Unit No. 2 Final Safety Analysis Report (FSAR) already take into account a  $\pm 3$  percent tolerance for the pressurizer and main steam safety valves. This is verified in Table 14.0.8-1 of the FSAR. As a result, the analysis provided with the proposed revision to the Technical Specifications was limited to the change to the analysis of the Steam Generator Tube Rupture Event. Upon approval of the proposed Technical Specification change, the new results for the Steam

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(1) J. F. Opeka letter to U.S. Nuclear Regulatory Commission, "Millstone Nuclear Power Station, Unit No. 2, Proposed Revision to Technical Specifications Pressurizer and Main Steam Safety Valves," dated September 29, 1995.

ADD 1/0

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Generator Tube Rupture Event will be incorporated into the FSAR.


The limiting reactor coolant system pressurization event for Millstone Unit No. 2 is the Loss of Load Event, as described in Section 14.2.1 of the FSAR. As stated in Section 14.2.8 of the FSAR, the Feedwater System Pipe Break Event is "not in the current licensing basis for Millstone Unit 2 and therefore is not analyzed."

There are no commitments contained within this letter. If there are any questions regarding this submittal, please contact Mr. Mario Robles at (860) 440-2073.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: J. F. Opeka  
Executive Vice President

BY:   
E. A. DeBarba  
Vice President

cc: T. T. Martin, Region I Administrator  
G. S. Vissing, NRC Project Manager, Millstone Unit No. 2  
P. D. Swetland, Senior Resident Inspector, Millstone Unit  
Nos. 1, 2, and 3