

## LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 58

REPORT 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

Surveillance testing of fire and smoke detectors per Technical Specification (T.S.) 4.19 was not completed within the surveillance interval including the  $\pm 25\%$  tolerance. All testing outside of the reactor building was completed during the interval, however, testing within the reactor building was not completed. The unit was shutdown because of unrelated events and the testing was completed per T.S. requirements. All detectors checked operable. This occurrence is reportable per T.S. 6.12.3.2.c.

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

SYSTEM CODE 9 10 CAUSE CODE 11 12 CAUSE SUBCODE 13 14 COMPONENT CODE 15 16 COMP SUBCODE 17 18 VALVE SUBCODE 19 20

LER/RO 17 REPORT NUMBER 18 19 EVENT YEAR 20 21 22 SEQUENTIAL REPORT NO. 23 24 25 26 OCCURRENCE CODE 27 28 29 REPORT TYPE 30 31 REVISION NO. 32 33

ACTION TAKEN 33 34 FUTURE ACTION 35 36 EFFECT ON PLANT 37 38 SHUTDOWN METHOD 39 40 HOURS 41 42 ATTACHMENT SUBMITTED 43 44 NPRD-4 FORM SUB 45 46 PRIME COMP. SUPPLIER 47 48 COMPONENT MANUFACTURER 49 50

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

The testing within the reactor building could not be completed within the T.S. surveillance time interval because radiation levels were too high to perform testing and remain consistent within ALARA radiation exposure considerations. The detectors tested satisfactorily at the next cold shutdown. A T.S. change was implemented in Amendment 54 of the licensee/T.S. which became effective on 3/23/81. This change provides for testing of the detectors inside the reactor building during each cold shutdown exceeding (cont'd)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

FACILITY STATUS 9 10 % POWER 11 12 OTHER STATUS 13 14 METHOD OF DISCOVERY 15 16 DISCOVERY DESCRIPTION 17 18

Surveillance Test

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

ACTIVITY RELEASED 9 10 CONTENT OF RELEASE 11 12 AMOUNT OF ACTIVITY 13 14 LOCATION OF RELEASE 15 16

NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PERSONNEL EXPOSURES NUMBER 9 10 TYPE 11 12 DESCRIPTION 13 14

NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PERSONNEL INJURIES NUMBER 9 10 DESCRIPTION 11 12

NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

LOSS OF OR DAMAGE TO FACILITY TYPE 9 10 DESCRIPTION 11 12

NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PUBLICITY ISSUED 9 10 DESCRIPTION 11 12

NA

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8408130526 840803  
PDR ADOCK 05000313  
S PDR

JE22  
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LICENSEE EVENT REPORT

EXHIBIT A

LER No. 50-313/80-014/03X-1

Occurrence Date: 04/15/80

Cause Description and Corrective Actions (Continued)

| 24 hours unless performed during the previous 6 months. It also requires testing during each refueling outage.



ARKANSAS POWER & LIGHT COMPANY  
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

August 3, 1984

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U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 1  
Docket No. 50-313  
License No. DPR-51  
Licensee Event Report  
No. 80-014/03X-1

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.2.c, attached is the subject report concerning failure to complete fire and smoke detector testing in the reactor building within the time period required by Technical Specifications. This is a revision to a previous submittal dated May 12, 1980.

Very truly yours,

John R. Marshall  
Manager, Licensing

JRM:RJS:ac

Attachment

cc: Mr. Richard C. DeYoung  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. Norman M. Haller, Director  
Office of Management & Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

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