

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) D. C. COOK PLANT UNIT-2										DOCKET NUMBER (2) 0 5 0 0 0 3 1 6										PAGE (3) 1 OF 0 2																																																						
TITLE (4) CONTROL ROD MISALIGNMENT																																																																										
EVENT DATE (5)									LER NUMBER (6)									REPORT DATE (7)									OTHER FACILITIES INVOLVED (8)																																															
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES												DOCKET NUMBER(S)																																			
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0 7			0 9			8 4			8 4			0 1			8			0 0			0 6			0 8			8 4															0 5 0 0 0																																
OPERATING MODE (9) 2															THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																																											
POWER LEVEL (10) 01013															20.402(b)															20.405(c)															50.73(a)(2)(iv)															73.71(b)														
															20.405(a)(1)(i)															50.36(c)(1)															50.73(a)(2)(v)															73.71(c)														
															20.405(a)(1)(ii)															50.36(c)(2)															50.73(a)(2)(vii)															OTHER (Specify in Abstract below and in Text, NRC Form 366A)														
															20.405(a)(1)(iii)															50.73(a)(2)(i)															50.73(a)(2)(viii)(A)																													
															20.405(a)(1)(iv)															50.73(a)(2)(ii)															50.73(a)(2)(viii)(B)																													
20.405(a)(1)(v)															50.73(a)(2)(iii)															50.73(a)(2)(x)																																												
LICENSEE CONTACT FOR THIS LER (12)																																																																										
NAME A. A. BLIND ENGINEERING DEPARTMENT SUPERINTENDENT															TELEPHONE NUMBER AREA CODE 6 1 6 4 6 1 5 - 1 5 1 0 1 1																																																											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																																										
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPRDS		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPRDS		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPRDS																																														
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SUPPLEMENTAL REPORT EXPECTED (14)																																																																										
YES (If yes, complete EXPECTED SUBMISSION DATE)															X NO															EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR																																		

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON 07-09-84 AT 1836 HOURS, DURING THE INITIAL STARTUP FOLLOWING A REFUELING OUTAGE, THE ROD POSITION INDICATION FOR RODS B-8 AND K-10 INDICATED GREATER THAN TWELVE (12) STEPS FROM THE GROUP DEMAND POSITION OF 228 STEPS. THE CONTROL ROD POSITIONS WERE CALCULATED BY SECONDARY COIL STACK VOLTAGE MEASUREMENTS AND IT WAS DETERMINED THAT B-8 WAS AT 214 STEPS AND K-10 WAS AT 215 STEPS INDICATING THAT BOTH RODS WERE MISALIGNED. THE EVENT WAS NON-CONSERVATIVE IN RESPECT TO TECHNICAL SPECIFICATION 3.1.3.1. THE APPLICABLE ACTION REQUIREMENT WAS MET AS THE UNIT WAS PLACED IN MODE 3 (HOT STANDBY).

ON 07-10-84 AT 1428 HOURS FOLLOWING ROD WITHDRAWAL, ALL CONTROL RODS WERE DETERMINED TO BE OPERABLE. THE RPI'S PREVIOUSLY EXCEEDING THE 12 STEP LIMIT, AS WELL AS OTHER RPI'S, WERE VERIFIED TO BE WITHIN ACCEPTABLE MARGINS BY SECONDARY COIL STACK VOLTAGE MEASUREMENTS. THE RODS WERE NOT BELIEVED TO BE MISALIGNED DUE TO VARIABLES AFFECTING THE SECONDARY COIL STACK VOLTAGES.

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PDR ADOCK 05000316
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)

D.C. COOK PLANT, UNIT-2

DOCKET NUMBER (2)

0500031684

LER NUMBER (5)

YEAR SEQUENTIAL REVISION
NUMBER NUMBER NUMBER

018-000

PAGE (3)

2 OF 2

TEXT (If more space is required, use additional NRC Form 385A's) (17)

ON 07-09-84 AT 1836 HOURS, DURING THE INITIAL STARTUP FOLLOWING A REFUELING OUTAGE, THE ROD POSITION INDICATION (RPI) FOR RODS B-8 AND K-10 INDICATED GREATER THAN TWELVE (12) STEPS FROM THE GROUP DEMAND POSITION OF 228 STEPS. THE LOW POWER PHYSICS TESTING THAT PRECEDED THIS EVENT HAD MOVED ALL THE CONTROL RODS FROM NEARLY FULL-IN TO NEARLY FULL-OUT POSITIONS. THE CONTROL ROD (IEEE COMPONENT FUNCTION IDENTIFIER = ROD) POSITIONS WERE CALCULATED BY SECONDARY COIL STACK VOLTAGE MEASUREMENTS. ROD B-8 WAS CALCULATED TO BE AT 214 STEPS AND ROD K-10 WAS CALCULATED TO BE AT 215 STEPS. THIS EVENT WAS NON-CONSERVATIVE IN RESPECT TO TECHNICAL SPECIFICATION 3.1.3.1, AS THERE WAS AN INDICATION THAT TWO (2) CONTROL RODS WERE MISALIGNED BY MORE THAN TWELVE (12) STEPS. THE APPLICABLE ACTION REQUIREMENT WAS MET AS THE UNIT WAS PLACED IN MODE 3 (HOT STANDBY). DUE TO THE CHANGING ELECTRICAL PROPERTIES OF THE PRESSURE HOUSING AND COIL STACKS FROM THERMAL CYCLING, THE SECONDARY VOLTAGES ARE UNSTABLE UNTIL THERMAL EQUILIBRIUM IS REACHED. THESE COMPONENTS HAD NOT REACHED THERMAL EQUILIBRIUM AT THE TIME THE VOLTAGE MEASUREMENTS WERE TAKEN. THIS IS BELIEVED TO BE THE CAUSE FOR THIS EVENT AS THE AC SECONDARY VOLTAGE READINGS TAKEN TO CALCULATE THE ROD POSITIONS DURING THIS EVENT HAD CHANGED FROM THE VALUES OBTAINED DURING RPI CALIBRATIONS.

ON 07-10-84, FOLLOWING ROD WITHDRAWAL, ALL RODS WERE DETERMINED TO BE WITHIN 12 STEPS OF GROUP DEMAND POSITION INDICATION. THE RPI'S PREVIOUSLY EXCEEDING THE 12 STEP LIMIT, AS WELL AS OTHER RPI'S WERE VERIFIED TO BE WITHIN ACCEPTABLE MARGINS BY SECONDARY COIL STACK VOLTAGE MEASUREMENTS.



INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
(616) 465-5901

August 8, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

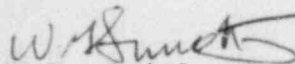
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Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10CFR50.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 84-018-0

Sincerely,


W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
J.G. Keppler, RO:III
M.P. Alexich
R.F. Kroeger
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